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Self-consistent cross-field transport model for core and edge plasma transport

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Abstract. A two-equation model to self-consistently determine cross-field fluxes in the edge and scrape-off layer region of diverted plasma is used to complete 2D mean-field edge transport description of plasma wall interaction. Inspired by the Reynolds Average Navier-Stokes simulations for neutral fluids, this model is based on the local evolution of the turbulent kinetic energy κ and its dissipation rate ε . These two equations are algebraically derived for RANS modeling and are very slightly modified and adapted to describe self-consistent plasma turbulent transport. The general features of the model are discussed and bridged to the well-known predator-prey and quasilinear models commonly used to investigate plasma transport. Specific closures are proposed based on the interchange turbulence. Results of the 1D model are confronted to experimental evidence by analyzing the computed SOL width and comparing the results to the existing scaling law for L-mode plasmas. Introducing a dependence on the shear of large scale flows, typically the zonal flows, 1D simulations can exhibit an H-mode like transition when increasing the input power, generating an increased stored energy thanks to an interface barrier located at the separatrix. Further 2D plasma-wall interaction simulations for WEST are analyzed that show a good match with the experimental profiles, as well as a ballooned transport driving turbulent transport in the divertor SOL and nearly no transport in the private flux region. The SOL width of WEST is also recovered. These results show the remarkable capability of the κ - ε model to capture key aspects of the physics of turbulent transport throughout the plasma knowing that a unique scalar free parameter is available to tune cross field transport in the whole 2D cross section of the plasma.

Introduction

The control of plasma-wall interaction in next step devices aiming at burning plasma operation is presently understood to be a key research topic bridging the physics of advanced divertor scenarios and technological constraints governing the heat flux exhaust capability of the wall components [1, 2]. The ITER divertor design is based on a long standing effort of transport simulations of the edge plasma in axisymmetric

geometry [3, 4]. This simulation effort has played a key role in defining guidelines for the divertor design. Recent effort has been made towards design optimization [5] and determination of reliable transport coefficients [6]. However, stepping towards high performance experiments near the operational limits, taking into account the aging of the components and experimental feedback, will require improved reliability of the numerical tools.

A suite of models is being developed, ranging from simplified models for optimization and uncertainty propagation to state-of-the-art first principle models of plasma turbulence transport in relevant plasma conditions. In that respect, full gyrokinetic simulations of the core and edge plasma are being used in the fusion community, but remain extremely costly from the computational point of view. In particular, the near-wall region requires addressing particle transport, hence electron and ion dynamics on the same footing, taking into account ionization particle sources and given magnetic as well as boundary condition geometries that are much more complex than those considered in the core [7, 8, 9]. Performing such simulations with 3D fluid codes that handle self-consistently all scales of the flow, from the grid spacing to device size, are generally still restricted to rather simplified geometries and only take into account a fraction of the atomic physics at play in plasma wall interaction, see [10, 11] and references therein.

Thus, in view of ITER operation or reactor design, the reduced models implemented in the *transport codes* are well adapted to provide relevant information on appropriate return times, similarly to the Reynolds Averaged Navier-Stokes (RANS) codes commonly used for engineering applications in the neutral fluid community [12]. These *transport codes* are based on reduced fluid models, generally assuming axisymmetry of the plasma and accordingly using axisymmetric averaged equations. The reduction of the number of degrees of freedom allows one to take into account additional equations describing surface physics processes and atomic physics in realistic tokamak geometries. There exists in the fusion community a certain number of such 2D state-of-the-art transport codes. The reference effort started with B2 [13], then leading to SOLPS [14, 15] and lately to SOLPS-ITER [16]. These three plasma transport modules are coupled to the kinetic code Eirene [17] for the neutral particle transport. The other major codes are UEDGE [18] (coupled to the DEGAS code [19] for neutral particles), SONIC [20], EDGE2D [21] (first coupled to the NIMBUS [22] for neutral particles and later to Eirene), and SolEdge2D-Eirene [11, 17, 23]. The latter, operated with configurations of most existing tokamaks in Europe and used in the present work, simulates the plasma edge and scrape-off layer (SOL) in a toroidally axisymmetric spatial domain including realistic wall geometry and detailed plasma-wall interaction. It is coupled to the Monte Carlo code Eirene [17], which generates the particle source from the various neutral ionization processes, for both atoms and molecules.

A major mathematical change in the structure of the transport equations is the step from microscopic convective turbulent transport to a larger scale diffusive transport. Models that are presently used for plasma-wall interaction assume that the transverse

fluxes are driven by local gradients, and consequently defined by diffusion or conductivity coefficients (equivalent to the turbulent eddy viscosity introduced in neutral fluids). The apparent universality of diffusive transport models is connected to the fact that it is in practice the simplest local transport model that can be implemented in equations. The effective particle diffusion D_n stemming from this process, and similarly the effective viscosity ν_n for the parallel momentum and effective heat conductivity χ_e and χ_i , respectively for the electrons and the ions, must be determined at each point of the mesh. As an example the perpendicular turbulent flux of particles $\tilde{n}\tilde{v}_\perp$, namely the convective transport of the density fluctuations \tilde{n} by the cross-field drifts \tilde{v}_\perp is closed as:

$$\langle \tilde{n}\tilde{v}_\perp \rangle_{cg} \sim -D_n \nabla_\perp \langle n \rangle_{cg} \quad (1)$$

where $\langle \rangle_{cg}$ stands for the coarse graining procedure. This step is most important since it determines in fact a projection operator and consequently the physics that one retains and that which is orthogonal [24, 25]. Ensemble averaging is often presented as the backbone for coarse graining. However, this elegant argument is not practical and not really informative regarding the rules that govern the coarse graining process. In practice an averaging operator over the high-frequencies is used; D_n is then an effective diffusion coefficient and $\nabla_\perp \langle n \rangle_{cg}$ the local transverse gradient of the coarse grained density field. For ITER, the global behavior of the plasma must be analyzed with time scales of a fraction of the energy confinement time in the core and a fraction of the thermal time scale of the plasma facing components, both in the range of seconds. The chosen coarse graining time scale τ_{cg} is therefore in the range of a second while the turbulence time scales τ_{turb} are usually assumed to range between microseconds and milliseconds. It is to be noted however that in flux driven turbulence simulations, no spectral gap is observed between this turbulence time range and the macroscopic time τ_{cg} [26]. This can be an issue in the coarse graining procedure [26]. Regarding space, two length scales are observed, the system size and the turbulence scale $\tilde{v}_\perp \tau_{turb}$. For magnetic fusion plasmas, the length $\tilde{v}_\perp \tau_{cg}$ exceeds the system size. In the plasma core, with system size given by the minor radius a , the regime $\tilde{v}_\perp \tau_{turb} \ll a$ can be assumed to hold. For plasma-wall interaction, characterized by the SOL width λ_n , the regime $\tilde{v}_\perp \tau_{turb} \approx \lambda$ is more likely. Defining and implementing a coarse graining procedure is then less straightforward [26].

In present transport simulations, the particle, momentum and heat diffusion coefficients are not determined self-consistently but are tuned to match experimental radial profiles usually known at a single poloidal location, typically in the midplane. This is done either by providing transport coefficients as input to the code, having checked a good match between the empirical profiles and the reconstructed ones, either in a more sophisticated manner by adjusting automatically these values according to the midplane profiles as part of the initial step of the run. In SolEdge2D-Eirene, the automatic fitting procedure based on a proportional-Integral feedback loop is implemented [27]. In both cases, radial profiles of the effective transport coefficients are determined and

it is assumed that these do not exhibit any other dependence that strongly limits the predictive capability of such codes. We remind that accurate values of the transport coefficients are required to determine the SOL width, which is the crucial parameter that governs plasma-wall interaction. Indeed, the competition between the SOL parallel conductive/convective transport and cross field transport combining large scale drifts, and turbulent and collisional transport [28] determines the width of the heat and particle channels impinging onto the target plates, and consequently the operation constraints governed by the exhaust limitations. It is also important to underline that in most transport codes used to investigate plasma-wall interaction, the local gradients, for example the gradient of the thermal energy ∇T are estimated in eV/m , irrespective of the magnetic geometry, while the transport coefficients are assumed to be constant on a given magnetic surface. Since the minimum distance between flux surfaces is found in the midplane on the low-field side, where the transport coefficients are determined, away from this location, the gradients then appear to be smaller due the magnetic field flux expansion. Consequently they yield a reduced flux. A ballooned transport is thus induced but not as a consequence of the known properties of micro-turbulence but because the diffusion and conductivity coefficients are assumed to be homogeneous on each flux surface. In such a framework, the induced ballooning is not governed by the physics of turbulent transport as reported in all tokamak configurations [29, 30, 31], but stems directly from the geometry of the magnetic surfaces.

Thus, these codes require stepping towards predictive transport modeling, freezing on a physics basis all the free parameters that account for cross-field plasma transport induced by the drifts [6]. Following our recent work in Ref. [32], we propose in this paper a model for the self-consistent estimation of cross-field fluxes in the edge and scrape-off layer regions of diverted plasma that has been implemented in SolEdge2D-Eirene. The present effort is inspired by the work done from the 70's in hydrodynamics [33], neutral fluid turbulence in contrast to plasma turbulence in the fluid framework, and adapted here to magnetically confined plasmas for fusion applications. In neutral fluids, the Boussinesq assumption that the turbulent stresses and the deformation speeds of the mean flow are proportional, defines the eddy viscosity ν_t . This key quantity defining the transport properties of these fluids is then related to the turbulence kinetic energy $\kappa \equiv \frac{1}{2}\langle \tilde{v}^2 \rangle$ and turbulence dissipation rate ε . We proposes here to follow a similar procedure, thus introducing evolution equations for κ and ε and from these deriving the dependence on space and time of the plasma transport. We will model this transport by a diffusive process, as done for neutral fluids: convective transport will also be discussed since since turbulent transport in the edge appears to be governed by bursts of ballistic events [34, 35, 36]. An approach similar to that addressed here was recently proposed by [37, 38], but in a simpler configuration for isothermal plasma and 2D closed field lines domain. In this latter work, the perpendicular transport coefficients are determined from the turbulent kinetic energy κ and the enstrophy which is an invariant in 2D turbulence in neutral incompressible fluids. Their equations are analytically derived from the interchange turbulence model implemented in the TOKAM2D code [34, 39]. This work

suggests that the connection between the κ - ε model and fundamental properties of turbulence, such as the energy circulation between fluctuations and mean field [40], could be investigated based on these results from plasma physics. It is not the scope of this paper to step in that direction, but rather to present the capability of such a model to describe the edge and SOL transport with self-organization of turbulent transport. The κ - ε model can be seen as an intermediate model reduction between that inspired by the quasilinear results [41, 42] and direct coupling of turbulence to mean-field transport [43, 44].

The paper is organized as follows. We first discuss the general features of the κ - ε framework and bridge this approach to predator-prey and quasilinear models that are used to investigate plasma transport. We then introduce the SolEdge2D-Eirene suite of codes in Section 2. The general advection diffusion transport equation for κ and ε , implemented in SolEdge2D-Eirene, are detailed and the rules used to close the system are presented. In the fourth section, a one-dimensional reduction is proposed to carry out fast scans of plasma parameters and determine the main trends of the solutions. In the fifth section, the numerical results are confronted to experimental data in two L-mode plasma discharges of the WEST tokamak [45]. Finally, a discussion and conclusion Section closes the paper.

1. Local κ - ε evolution, comparison to existing turbulent transport models

1.1. The κ - ε transport model

In this work, we consider the standard κ - ε model used in neutral fluids [33] where κ is the kinetic energy per unit mass of the fluctuating transverse velocity, typically $\approx \langle \tilde{v}_E^2 \rangle$ and ε is a damping process acting on κ . One thus retains the same evolution equations.

$$\partial_t \kappa + \nabla \cdot \mathbf{K} = S_\kappa - \varepsilon \quad (2a)$$

$$\partial_t \varepsilon + \nabla \cdot \mathbf{E} = S_\varepsilon \quad (2b)$$

Here $\nabla \cdot \mathbf{K}$ is the divergence of the flux of κ and $S_\kappa - \varepsilon$ is the local evolution term of κ . The evolution equation for ε (2b) is built to be similar to that of κ . The term $\nabla \cdot \mathbf{E}$ accounts for the divergence of the flux of ε and S_ε is the local evolution term of ε . The coupling between these equations that is made explicit by the term $-\varepsilon$ on the right hand side of (2a) identifies the field ε as a rate of turbulent energy dissipation and determines the relative normalization of ε with respect to κ .

The source terms S_κ and S_ε are expanded up to order 2: $S_\kappa = \gamma_\kappa \kappa - \beta_\kappa \kappa^2$, $S_\varepsilon = \gamma_\varepsilon \varepsilon - \beta_\varepsilon \varepsilon^2$. The linear part depends on the growth rates γ_κ and γ_ε . The non-linear parts are chosen to be restoring terms with β_κ and β_ε positive. The evolution equations (3) for two fields κ and ε in \mathbb{R}^+ exhibit therefore a generic local form. Assuming diffusive transport in the transverse direction and the expansion of S_κ , S_ε described above, the system (2)

takes the following form:

$$\partial_t \kappa - \nabla_{\perp} \cdot (D_{\kappa} \nabla_{\perp} \kappa) + \nabla_{\parallel} \cdot \mathbf{K}_{\parallel} = \gamma_{\kappa} \kappa - \beta_{\kappa} \kappa^2 - \varepsilon \quad (3a)$$

$$\partial_t \varepsilon - \nabla_{\perp} \cdot (D_{\varepsilon} \nabla_{\perp} \varepsilon) + \nabla_{\parallel} \cdot \mathbf{E}_{\parallel} = \gamma_{\varepsilon} \varepsilon - \beta_{\varepsilon} \varepsilon^2 \quad (3b)$$

It can also be shown that equation (3b) can be rewritten as a Ginzburg-Landau amplitude equation [46, 47] for the amplitude \mathcal{A} such that $\varepsilon = |\mathcal{A}|^2$. This further underlines the generic structure of the κ - ε system (3). In the two following Sections, we discuss the connection of the κ - ε approach to models presently used to investigate plasma turbulence: the predator-prey and quasilinear transport models.

1.2. Local κ - ε dynamics and comparison to predator-prey models

Let us consider here the local evolution equations for the fields κ and ε , hence system (3) without cross-field and parallel transport.

$$\partial_t \kappa = \gamma_{\kappa} \kappa - \beta_{\kappa} \kappa^2 - \varepsilon \quad (4a)$$

$$\partial_t \varepsilon = \gamma_{\varepsilon} \varepsilon - \beta_{\varepsilon} \varepsilon^2 \quad (4b)$$

One can consider the system (4) as a predator-prey system and more generally any kind of reservoir system with internal dynamics and coupling [48, 49]. The predator-prey dynamics [50] have been introduced in plasma-physics to investigate bifurcation-like phenomena of transport properties [51, 52]. However, rather than adjusting the predator-prey model to match a particular behavior, we only consider in this paper the local evolution terms defined by the κ - ε model (3) as chosen for neutral fluids [33]. The field ε is an effective predator of the prey κ . Indeed the two species exhibit growth rates that are similar and do not depend on either species κ or ε . The effective predator ε does not feed directly on κ . Rather the existence of ε inhibits the prey development as if modifying the ecosystem in an unfavorable way for κ . The fixed points of the predator equation are $\varepsilon = 0$ and $\varepsilon = \gamma_{\varepsilon}/\beta_{\varepsilon}$. For $\beta_{\varepsilon} > 0$, one thus finds a phase transition behavior such that for $\gamma_{\varepsilon} < 0$, $\varepsilon = 0$ is the only possible fixed point while for $\gamma_{\varepsilon} > 0$ two fixed points exist. One obtains similarly that for $\gamma_{\kappa} < 0$ the only fixed point is $\kappa = 0$. This transition for the coupled equations is reminiscent of the bifurcation in the Ginzburg-Landau amplitude equation such that for negative growth rate the stable fixed point is zero that becomes unstable when the growth rate is positive.

The comparison to the predator-prey models provides a first understanding of the field ε as the predator of the turbulent energy. One can also remark that ε is akin to the time derivative of κ . The set of equations can then be understood as the first two of a hierarchy of equations coupling increasing order time derivatives. The closure at ε is performed at the same order as the Newtonian closure in mechanics. The fact that κ ε belong to such a hierarchy of time derivatives justifies the choice to retain similar equations for both κ and ε . One can conjecture that this structure is the key point that drives the efficiency of the κ - ε model. Let us now address the properties

of the non-linear saturation terms. In neutral fluids [33] the definitions of β_κ and β_ε are guided by dimensionality arguments. Hence $\beta_\kappa = 1/D_\omega$ where D_ω is a diffusion coefficient. Regarding β_ε we note that ε is dimensionally equivalent to the rate of energy transfer between scales that governs the cascade paradigm of steady-state turbulent spectra. Compared to the choice made for neutral fluids, we then make one step towards interpreting the κ - ε model. The field κ stands for the turbulent energy of the modes that govern turbulent transport. In the framework of turbulent cascades, ε is a dissipation mechanism that can be understood as the rate of energy transfer by direct and inverse energy cascade [12] to regions of the spectrum that weakly contribute to transport. In the generic case of a direct cascade ε couples this energy to the Kolmogorov region of the spectrum where energy is dissipated. This interpretation in terms of a direct energy cascade is not mandatory for the present work, we believe that it provides an understanding of the physics taken into account by ε . It is to be noted that it is often argued that 2D turbulence leads to an inverse energy cascades. This argument is too loose and does not hold when considering turbulent transport, namely turbulent convective transport of density, pressure, etc. coupling the turbulent velocity to fluctuations of the transported field at all scales. In this case the existence of a second quadratic invariant akin to the enstrophy is possible [53] but not guaranteed. Lastly, the parallel transport in the third dimension cannot be ignored, especially the parallel losses that characterize the SOL physics, and can break the symmetry conditions required in neutral fluid 2D turbulence for a second quadratic invariant to exist. For steady state conditions at a given scale ℓ , a dimensional analysis readily yields the relationship between the rate of turbulent energy transfer ε and the turbulent energy κ .

$$\kappa \propto \ell^{2/3} \varepsilon^{2/3} \quad (5)$$

This universal feature drives both direct and inverse energy cascades. Stepping to determining the power law dependence in the region of the spectrum governed by the energy cascade (direct as well as inverse), one recovers the well known power law $\ell^{5/3} \varepsilon^{-1/4}$ of the Kolmogorov theory [54].

To enforce the steady-state relation of turbulent spectra (5) in the κ - ε model, we set $\beta_\varepsilon \propto \kappa^{-3/2}$. This is a slight change with respect to the standard neutral fluid κ - ε model where $\beta_\varepsilon \propto \kappa^{-1}$ [33]. For the sake of generality, we shall also consider $\beta_\varepsilon \propto \kappa^{-\eta}$, which depending on η encompasses the choice made for neutral fluids $\eta = 1$ and that made in (5), $\eta = 3/2$. It will be shown that all values of $\eta \geq 1$ exhibit comparable properties, the choice of $\eta \geq 1$ does not introduce critical changes to the model. The fixed points of the local evolution for ε is then zero and $\varepsilon = \gamma_\varepsilon / \beta_\varepsilon = \gamma \kappa_* (\kappa / \kappa_*)^{3/2}$, $\gamma > 0$ is a proportionality factor, dimensionally the inverse of a time, and $\kappa_* \in \mathbb{R}^+$ is a convenient normalization of κ to be chosen hereafter. The control parameter γ governs the amplitude of the dissipation via the energy cascade for a given turbulent energy. The effect of varying this control parameter is addressed in the following. The steady state non-trivial fixed point thus leads to $\kappa \propto \varepsilon^{2/3}$ as in (5). Given $\varepsilon = \gamma \kappa_* (\kappa / \kappa_*)^{3/2}$,

the fixed point for κ is then determined by the equation:

$$\gamma_\kappa \kappa - \frac{1}{D_\omega} \kappa^2 - \frac{\gamma}{\kappa_*^{1/2}} \kappa^{3/2} = 0 \quad (6a)$$

Solutions other than $\kappa = 0$ are thus determined by:

$$\frac{\kappa}{\kappa_*} + \left[\left(\frac{\kappa}{\kappa_*} \right)^{1/2} - \frac{\gamma_\kappa}{\gamma} \right] \frac{\gamma D_\omega}{\kappa_*} = 0 \quad (6b)$$

More generally, we set $\varepsilon = \gamma \kappa_* (\kappa/\kappa_*)^\eta$, with $\eta = 1$ for standard neutral fluids, and $\eta = 3/2$ for (5). We define $\kappa_* = |\gamma_\kappa| D_\omega > 0$ so that the fixed point is either $\kappa/\kappa_* = 0$ or the positive root of:

$$\frac{\kappa}{|\kappa_*|} + \frac{\gamma}{|\gamma_\kappa|} \left(\frac{\kappa}{\kappa_*} \right)^{(\eta-1)} - \frac{\gamma_\kappa}{|\gamma_\kappa|} = 0 \quad (6c)$$

For the two reference values of η , $\eta = 1$ and $\eta = 3/2$, analytical solutions other than $\kappa = 0$ are obtained:

$$\eta = 1 \quad ; \quad \kappa = \gamma_\kappa D_\omega \quad (7a)$$

$$\eta = \frac{3}{2} \quad ; \quad \kappa^{1/2} = \left(-\frac{\gamma}{2|\gamma_\kappa|} + \left[\left(\frac{\gamma}{2|\gamma_\kappa|} \right)^2 + \frac{\gamma_\kappa}{|\gamma_\kappa|} \right]^{1/2} \right) (|\gamma_\kappa| D_\omega)^{1/2} \quad (7b)$$

For fixed $\gamma > 0$, and varying the control parameter γ_κ , one finds, as with the standard Ginzburg-Landau wave amplitude equation, that for $\gamma_\kappa < 0$ the only positive fixed point is $\kappa = 0$. Conversely, for $\gamma_\kappa \geq 0$ the stable fixed point is either $\kappa = 0$ or the solution given by (6c). Let us now assume that $\gamma_\kappa > 0$ is fixed and vary γ that governs the coupling between the two fields κ and ε via the damping process at play in the spectrum, Figure 1. For $\gamma = 0$, $\varepsilon = 0$ and the two fields are decoupled, $\kappa/\kappa_* = 1$. For the relevant values $\eta \geq 1$, as γ is increased, κ decreases monotonically from its maximum value κ_* towards zero, Figure 1 left hand side, while ε first increases with γ before decaying towards zero when $\gamma \rightarrow +\infty$, the maximum being achieved for $\gamma/\gamma_\kappa = 0.5^{2-\eta}$ and $\varepsilon/\varepsilon_* = 0.25$ with $\varepsilon_* = \gamma_\kappa \kappa_*$, Figure 1 right hand side.

Compared to the case without coupling, readily obtained by setting $\gamma = 0$ so that $\varepsilon = 0$, adding the damping by ε in the equation for the turbulent energy, introduces a regulation with a possible time delay because this damping is governed by an independent equation with different time scales. However, the latter effect will not occur if the time scales that govern the evolution of ε are small, hence $\gamma \rightarrow +\infty$ so that ε exhibits an adiabatic response. Regarding the dependence on η , there is a change in behavior between $\eta < 1$ open circles, and $\eta \geq 1$ closed symbols, as shown on Figure 1. In particular $\kappa \rightarrow 0$ when $\gamma \rightarrow 0$ for $\eta < 1$, while $\kappa/\kappa_* \rightarrow 1$ for $\eta \geq 1$. Furthermore, instead of obtaining an asymptotic convergence towards zero when the control parameter γ is increased as with $\eta > 1$, for $\eta = 1$ both κ and ε switch to zero above the critical value $\gamma = \gamma_\kappa$ with a discontinuity of their derivative. The value $\eta = 1$ used in the standard κ - ε model is a marginal value with respect to the behavior for $\eta > 1$. It also appears that the

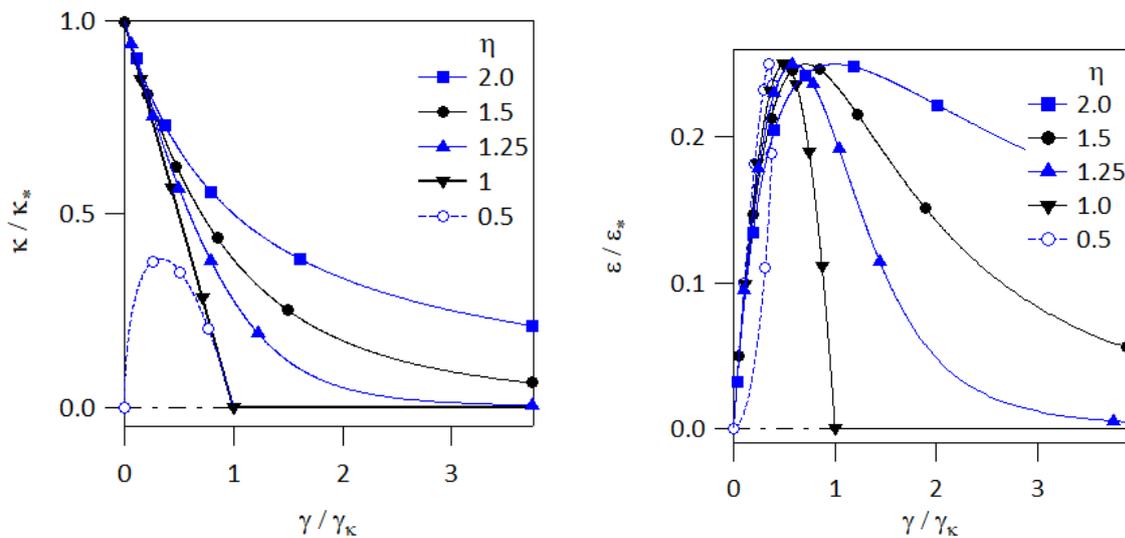


Figure 1. Variation of κ (left hand side) and ε (right hand side) with the control parameter γ , for different values of the exponent η : $\eta = 1$ black line head down closed black triangles, $\eta = 3/2$ black line closed black circles, $\eta = 1.25$ blue line closed head up blue triangles, $\eta = 2$ blue line closed blue triangles and $\eta = 0.5$ dashed blue line open blue circles. Reference values $\eta = 1$, RANS $\kappa - \varepsilon$ model [12] and $\eta = 3/2$ Kolmogorov theory [54].

cases $\eta \geq 1$ are qualitatively comparable so that the particular choice of $\eta > 1$ does not determine specific properties of the fixed points of the local dynamics.

It is to be underlined that the present version of the predator-prey model implemented in this κ - ε model is not chosen to exhibit particular physics in terms of bifurcations, limit cycles etc. Although it is convenient to see in the loss term ε that governed by the losses at the Kolmogorov dissipation scale via the energy cascade this is not the unique loss channel. Indeed, in the fusion plasma literature, an important loss path controlling turbulent transport is governed by a coupling to large scale flows [55, 56, 57]. This result fits with the picture of an inverse energy cascade although disparate wave coupling is often considered to be in fact the main drive. The latter can be understood as an alternative loss channel described by ε . In the following such a feature is taken into account by a modification of the ε local evolution. Indeed, it is understood that either the nonlinear coupling of the source term to the free energy, hence the dependence of γ_κ and γ_ε on the various gradients [34], or the shearing effect via the self generated zonal flow [55, 56, 57] are the main players in the turbulence energy evolution, something of the form of ε but governed by different dynamics [52]. In this context, it is also important to mention the approaches based on prey-predator models [58] and used to investigate the H-mode transport barrier, its onset as well as its dynamics in the vicinity of the threshold. The aspect that is considered there, as well as in [52], is that the control mechanism of the kinetic energy κ of the quasi-2D plasma turbulence is governed by large scale and meso-scale processes such as the $\mathbf{E} \times \mathbf{B}$ shearing of turbulence driven by zonal flows and mean flows [55, 56, 57]. In this framework, ε , the predator for

turbulence [58], stands for the stabilizing shearing effect of these large scale flows. The more complex local dynamics for κ - ε govern the features that are reminiscent of the L-H transition, as reported in [58, 59]. The comparison of the κ - ε model to the predator-prey models used to describe plasma self-organization opens the way to further adapt the standard κ - ε model by modifying the local evolution. We leave this investigation to future work. However, we show in this paper that including a shearing effect in the standard framework, without complex and highly nonlinear dynamics, already provides a scheme to self-consistently generate an interface barrier, Section 3.4.

1.3. κ - ε transport model and quasilinear framework

In the framework of reduced models, in particular for real time control, a renewed interest has been given to the quasilinear theory [60, 61, 62] by using a large data base of first principle simulations and experiments to constrain the quasilinear model [63, 42, 64]. We show here that the κ - ε model can be seen as an extension of the quasilinear theory, determining consistently, at each position and each time, the turbulent kinetic energy, typically κ , and the characteristic time governing the width of the resonance. Let us now consider a simplified quasilinear model for plasma transport governed by the following generic equation:

$$\partial_t f + \mathcal{L}f = S \tag{8}$$

We assume that one can define an averaging procedure such that f_0 is the projection of f with this averaging procedure, and such that the projection of $f_1 = f - f_0$ is zero, hence $\langle f \rangle = f_0$ and $\langle f_1 \rangle = 0$. The averaging projection can also be applied on the evolution operator $\mathcal{L} = L_0 + L_1$ with $\langle \mathcal{L} \rangle = L_0$ and $\langle L_1 \rangle = 0$. For simplicity we assume $\langle S \rangle = S$. Equation (8) can then be split into a set of two equations, for the mean and the fluctuations:

$$\partial_t f_0 + L_0 f_0 + \langle L_1 f_1 \rangle = S \tag{9a}$$

$$\partial_t f_1 + L_0 f_1 + L_1 f_0 + L_1 f_1 - \langle L_1 f_1 \rangle = 0 \tag{9b}$$

The quasilinear theory simplifies (9) by dropping the non-linear fluctuation terms in (9b) hence removing $L_1 f_1 - \langle L_1 f_1 \rangle$ while retaining $\langle L_1 f_1 \rangle$ in (9a). One could justify this step by considering some ordering such that the mean is of order 0 while fluctuations are of order 1, however, this does not imply that L_1 applied to f_1 is of order 2. This is clear with the example $L_1 = v_1 \nabla_\perp$, v_1 and f_1 being of order 1, but one cannot state that $\nabla_\perp f_1$ is of order 1 and not order 0. Furthermore, should $L_1 f_1$ be of order 2, it can be questionable to neglect terms of order 2 in an equation where the other terms are of order 1 as in (9b) while retaining an order 2 term in (9a) where the other terms are of order 0. It is more interesting here to consider symmetries. Indeed the term $L_1 f_1 - \langle L_1 f_1 \rangle$ can be seen as a correction to $L_0 f_1$, and considering that f_1 does not belong to the kernel of L_0 , this correction does not change the structure of (9b). The

latter can then be rewritten as:

$$\partial_t f_1 + L_0^* f_1 + L_1 f_0 = 0 \quad (10a)$$

$$L_0^* f_1 = L_0 f_1 + L_1 f_1 - \langle L_1 f_1 \rangle \quad (10b)$$

$$f_1 = -\frac{1}{\partial_t + L_0^*} L_1 f_0 \quad (10c)$$

The formal solution (10c) is obtained by assuming that the operator $\partial_t + L_0^*$ can be inverted implying in particular that f_1 does not belong to the kernel of $\partial_t + L_0^*$. One can then consider the transport equation for the mean f_0 :

$$\partial_t f_0 + L_0 f_0 - \left\langle L_1 \frac{1}{\partial_t + L_0^*} L_1 \right\rangle f_0 = S \quad (11)$$

Let us now discuss this result in the framework of transport in a magnetized plasma, typically setting $L_0 = v_{\parallel} \nabla_{\parallel}$ and $L_1 = v_{\perp} \nabla_{\perp}$ where the subscript \parallel and \perp refer to the direction of the magnetic field. We thus focus in this discussion on the position dependence of f_0 with respect to the magnetic surfaces; ∇_{\parallel} depending on the variation within a magnetic surface, and, ∇_{\perp} depending on the variation between magnetic surfaces, hence $\nabla_{\perp} \approx \nabla \psi \partial_{\psi}$ where ψ is a magnetic surface label. Setting $L_1 = 0$ and $S = 0$ in (11), and given the symmetry $\nabla_{\parallel} \psi = 0$, f_0 is a steady state solution when f_0 belongs to the kernel of L_0 , hence f_0 only depends on ψ . In this case perfect confinement is obtained, the source term S is not needed. Breaking the symmetry with fluctuations, $L_1 \neq 0$ then governs the need for a source term to achieve a steady state. With the proposed operators, we have implicitly assumed the simplification $\nabla_{\perp} v_1 = 0$, furthermore, approximating L_0^* by L_0 as done in the quasilinear framework, one obtains:

$$\partial_t f_0 + L_0 f_0 - \nabla_{\perp} \left\langle \frac{v_1^2}{\partial_t + v_{\parallel} \nabla_{\parallel}} \right\rangle \nabla_{\perp} f_0 = S \quad (12a)$$

$$D_{ql} = \left\langle \frac{1}{2} v_1^2 \tau_{ql} \right\rangle = \left\langle \kappa \tau_{ql} \right\rangle \quad (12b)$$

$$\tau_{ql} = \frac{2}{\partial_t + v_{\parallel} \nabla_{\parallel}} \quad (12c)$$

The quasilinear analysis thus yields three important features, (i) one obtains a diffusive like transport in the cross-field direction, (ii) the diffusion coefficient is proportional to κ , and (iii) a characteristic time is required to completely determine the diffusion coefficient D_{QL} (12b). It is interesting to note that the result can be extended to the general case hence without approximating L_0^* by L_0 . This can modify some aspects of the commutation between v_1 and the operator τ_{ql} but does not change the main features of the result, in particular the diffusive structure. Two key assumptions constrain the validity of this result, first the possibility of defining the averaging procedure in line with the symmetries that govern the operator \mathcal{L} and second that the inversion of either $\partial_t + L_0$ or $\partial_t + L_0^*$ is possible, and, when possible, does not govern a change in the structure

of the final result such as the explicit dependence on $\nabla_{\perp} f_0$. Closing the quasilinear approach then requires to determine both the proper κ and τ_{ql} . In the most recent effort, this is achieved using a large data base of local gyrokinetic simulations completed by setting a proportionality factor using experimental evidence [42, 64].

The dimensional argument used for the κ - ε model can be considered to determine the time τ_{ql} since the ratio κ/ε has the dimension of a time, one can then set $\tau_{ql} \propto \kappa/\varepsilon$ and one obtains therefore:

$$D_{ql} = C_{ql} \frac{\kappa^2}{\varepsilon} \quad (13)$$

where C_{ql} is a proportionality factor. It is also interesting to note that the time κ/ε also appears in the evolution of κ (3a).

$$\frac{1}{\kappa} \partial_t \kappa - \frac{1}{\kappa} \nabla_{\perp} \cdot (D_{\kappa} \nabla_{\perp} \kappa) + \frac{1}{\kappa} \nabla_{\parallel} \cdot \mathbf{K}_{\parallel} = \gamma_{\kappa} - \beta_{\kappa} \kappa - \frac{\varepsilon}{\kappa} \quad (14)$$

Indeed, the local evolution, right hand side of (14) depends on three characteristic time scales, the linear drive $1/\gamma_{\kappa}$, the non-linear restoring time D_{ω}/κ and κ/ε . The relations we have obtained at that stage appear to be comparatively complicated because we have chosen the proportionality factor γ as free parameter. When considering as free parameter the characteristic time $\tau_{ql} = \kappa/\varepsilon$, one obtains:

$$\gamma_{\kappa} \tau_{ql} = \gamma_{\kappa} \frac{\kappa}{\varepsilon} = \frac{\gamma_{\kappa}}{\gamma} \left(\frac{\kappa}{\kappa_*} \right)^{-(\eta-1)} \quad (15)$$

Most of the complexity of this system is thus governed by the relationship between the time τ_{ql} and κ . Conversely, the steady state solutions exhibit simple dependencies in terms of $\gamma_{\kappa} \tau_{ql}$, Figure 2:

$$\frac{\kappa}{\kappa_*} = 1 - \frac{1}{\gamma_{\kappa} \tau_{ql}} \quad (16a)$$

$$\frac{\varepsilon}{\varepsilon_*} = \frac{1}{\gamma_{\kappa} \tau_{ql}} \left(1 - \frac{1}{\gamma_{\kappa} \tau_{ql}} \right) \quad (16b)$$

One finds here that $1/(\gamma_{\kappa} \tau_{ql}) \leq 1$, hence $\tau_{ql} \geq 1/\gamma_{\kappa}$ so that τ_{ql} is the longest time scale in the system. In this framework, setting $D = \kappa^2/\varepsilon$, one finds a very simple expression for the diffusion coefficient:

$$\frac{D}{D_*} = \frac{\kappa^2 \varepsilon_*}{\varepsilon \kappa_*^2} = \gamma_{\kappa} \tau_{ql} - 1 \quad (16c)$$

The form (13) is the reference one for the diffusion transport coefficients in the κ - ε framework. Therefore, one can consider that the κ - ε model is an extension of the quasilinear framework such that the fields κ and ε are determined within the local transport model, and evolved self-consistently rather than being imported using other tools, either fitting of experimental evidence or determined using a data base of local gyrokinetic simulations. Regarding plasma-wall interaction, which exhibits at least

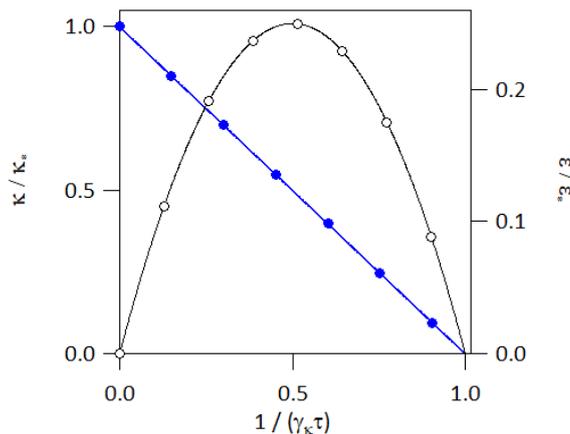


Figure 2. Variation of κ blue line closed circles left hand side axis, and ε black line open circles, right hand side axis, with the control parameter $\gamma_\kappa \tau_{ql}$.

a 2D dependence in space, $2D$ measurements of the gradients are out of reach, the κ - ε approach then appears as an extension of the quasilinear theory that is better suited to address the complexity and variability in geometry and parameter space of the problem. Furthermore, rather than fitting the characteristic time, the κ - ε model allows one to determine self-consistently this time and in particular the dynamics with the possible occurrence of time delays in the response of ε .

2. Closure of the κ - ε model for plasma transport

2.1. The κ - ε model with diffusive cross-field transport

In Section 1.2 we have discussed the form taken by the non-linear restoring terms of the local evolution. For convenience we now write the restoring term as $\beta_\varepsilon = V/\kappa^{3/2}$ where V is a free parameter, dimensionally equivalent to a velocity. Furthermore, as in (3) we neglect the large scale flows that arise from the non-homogeneity of the magnetic field as well as from large scale electric fields. This is in line with standard modeling of plasma-wall interaction but will fall short whenever a finer description of cross-field transport is required. In particular when contemplating the κ - ε model as a tool to investigate the properties of turbulent transport described by first principle fluid or gyrokinetic simulations. To reduce the number of free parameters, we impose in the present work $D_\kappa = D_\varepsilon = D$ where $D = \kappa^2/\varepsilon$. Further work and comparison to transport data bases could lead to introducing further control parameters to ensure a better match with experimental and simulation evidence. Taking into account these different changes, one then obtains:

$$\partial_t \kappa - \nabla_\perp \cdot (D \nabla_\perp \kappa) + \nabla_\parallel \cdot \mathbf{K}_\parallel = \gamma_\kappa \kappa - \frac{\kappa^2}{D_\omega} - \varepsilon \quad (17a)$$

$$\partial_t \varepsilon - \nabla_\perp \cdot (D \nabla_\perp \varepsilon) + \nabla_\parallel \cdot \mathbf{E}_\parallel = \gamma_\varepsilon \varepsilon - V \frac{\varepsilon^2}{\kappa^{3/2}} \quad (17b)$$

The transverse transport that governs the plasma fields is also assumed to be diffusive, and the coefficients proportional to $D = \kappa^2/\varepsilon$. Various boundary conditions have been implemented for κ and ε . For the simulations presented in this paper the radial boundary conditions are set by vanishing second derivatives. A similar boundary condition has been used in the parallel direction for the 2D SolEdge simulations. This choice does not constrain the outflux and boundary value. Several 1D simulations have been run with vanishing boundary values for both κ and ε . The change only modifies a thin layer in the outer SOL, in agreement with the importance of the local dynamics for κ and ε compared to transport.

A first closure issue consists in determining these various diffusion coefficients. we first assume that the turbulent (subscript t) fluid viscosity ν_t is proportional to D with proportionality factor \mathcal{C}_ν . The relevant turbulent particle diffusion D_n is then defined by the turbulent Schmidt number $Sc_t = \nu_t/D_n$, while the electron and ion turbulent heat diffusivity are determined by a single turbulent Prandtl number Pr_t , hence:

$$\nu_t = \mathcal{C}_\nu \frac{\kappa^2}{\varepsilon} \quad ; \quad D_n = \frac{\nu_t}{Sc_t} \approx \nu_t \quad ; \quad \chi_e = \chi_i = \frac{\nu_t}{Pr_t} \approx 2\nu_t \quad (18)$$

The constant parameter \mathcal{C}_ν is to be determined with an appropriate constraint, typically a fitting procedure of either empirical or simulation data. In the present work, we further assume, as rather standard in neutral fluids Ref.[65] and references therein, that the turbulent Schmidt number $Sc_t = \nu_t/D_n$ is of order unity. We shall also consider an empirical observation from midplane profiles which have been matched by steady state transport simulations with the constraint $\chi_{e,i} = 2 \times D_n$ [66]. Accordingly, we shall set the turbulent Prandtl number $Pr_t = \nu_t/\chi$ to be equal to 1/2. Again future work with fitting procedures using empirical or simulation data should be used to provide a firm basis to set these closure parameters, possibly different from those chosen in this paper.

2.2. Growth rates closure for the plasma κ - ε model

Since κ and ε are determined locally in space and at each time step by the model (3), the diffusion coefficient D determined from κ and ε is also defined locally and exhibits an evolution in time. Consistently, we consider therefore that the growth rates in (3) or (17) are local and time dependent. This is a major particularity of the κ - ε model in neutral fluids that we transpose to plasmas. Plasma turbulence in magnetically confined plasmas mostly arises from instabilities usually identified according to their linear drive. For electrostatic turbulence two main classes can be identified: (i) interchange-like instabilities governed by the magnetic field inhomogeneity that drives a current across the magnetic surfaces, (ii) the drift wave instability mostly driven by parallel currents. The interchange-like mechanism drives Ion Temperature Gradients [67] and SOL-interchange [68]. It is characterized by a poloidal asymmetry of the drive, leading to turbulence ballooning on the low field side of the torus. Conversely the drift wave instability [69, 70], including the sheath negative resistivity mechanism [71], does not exhibit a poloidal asymmetry in the drive. However, in the non-linear stage working

back to a particular mechanism is questionable [70]. Furthermore, it has been found that linearly unstable mechanisms can "cooperate" [72], or compete, one instability inhibiting the other [73]. Such predator-prey behavior could be transposed to κ - ε models. This is left for future work. For the present purpose of using the κ - ε model to investigate plasma-wall interaction, we have decided to single out the interchange-like instability driven by the plasma pressure gradient. As discussed, such a mechanism is understood to drive the poloidal asymmetry observed in the turbulent transport between the low field side and the high field side, the so-called ballooning of the turbulence, which is not consistently taken into account in 2D transport codes addressing plasma-wall interaction. The motivation is to recover this imbalance and does not mean that we regard the specific interchange instability as the only player. As in the standard κ - ε model, we shall furthermore consider the simplification $\gamma_\kappa = \gamma_\varepsilon$, so that $\gamma_\kappa = \gamma_\varepsilon = \gamma_I$ where γ_I is the linear interchange instability growth rate [68, 34, 39]. The case with $\gamma_\kappa \neq \gamma_\varepsilon$ is also of interest and addressed elsewhere [74].

$$\gamma_I = \frac{c_s}{R} \sqrt{R^2 \frac{\nabla p \cdot \nabla B_T}{p B_T}} \quad (19)$$

The variation in space of the axisymmetric toroidal magnetic field B_T is governed by the tokamak major radius R . The time scale R/c_s , where c_s is the sound velocity, is chosen consistently with a drive depending on the pressure gradient ∇p . The pressure p and its gradient play a key role since they vary in space and time while $R B_T$ is constant when assuming axisymmetry. More complete models can be addressed, either by assuming competing growth rate of the different instabilities that add up to drive the same turbulent energy κ , or competing κ - ε systems, one for each instability, such that one would then add the various diffusion coefficients. However, the implicit rule in (19), namely that $\gamma_I = 0$ when $(\nabla p/p) \cdot (\nabla B_T/B_T) \leq 0$ governs a ballooning of this local drive. It is then possible that a class of instabilities characterized by such a ballooning feature can be addressed with this particular growth rate. As will be shown in Section 2.6, a vanishing drive combined to the other transport properties will govern an effective damping of the turbulent energy. The overall effect is then reminiscent of the growth rates derived in Refs.[34, 39, 68] that take into account the transport features.

2.3. Ballooned transport in the poloidal plane

In order to check that the growth rate (19) allows one to recover the salient features of the ballooned transport patterns in the poloidal plane, we analyze various simulations of a particular TCV experiments [75, 76] performed with the 3D turbulence code TOKAM3X [10] and SolEdge2D-EIRENE. Depending on the simulations, ballooning effects are either included self-consistently, as with TOKAM3X and SolEdge2D with the κ -only transport model, or enforced in the transport model of SolEdge2D, or ignored as with standard SolEdge2D transport model. Let us first analyze the SolEdge2D-EIRENE simulations without self-consistent turbulent transport from Ref.[75], Figure 3. On panel (a), we have reproduced the particle flux as obtained with the procedure of midplane

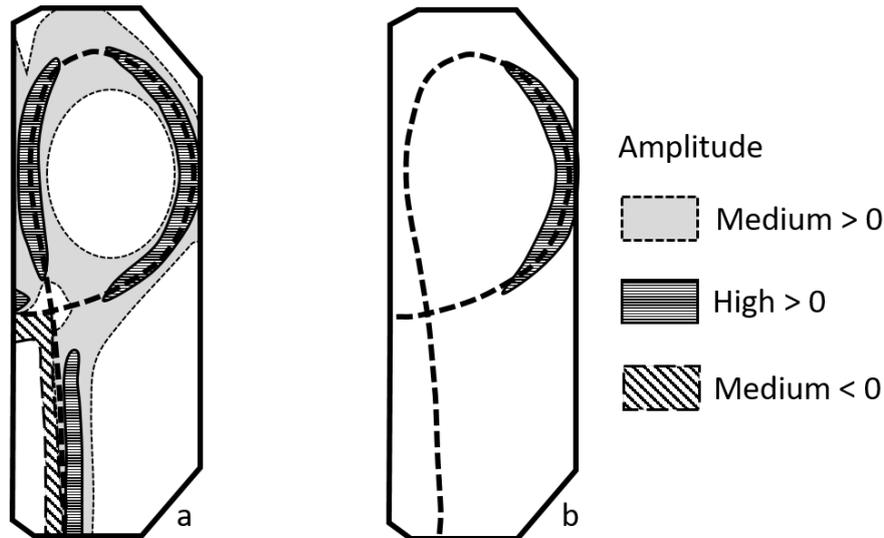


Figure 3. Sketch of the 2D transport pattern in TCV in the large X-point height configuration. Pattern of the particle flux in the poloidal plane, simulations with SolEdge2D-EIRENE but without self-consistent turbulent transport, sketch from Ref.[75]. Panel *a*: standard transport modeling in 2D plasma-wall interaction. Panel *b*: modified choice of transport modeling enforcing ballooned transport. Strong outward particle flux is labeled "High > 0", moderate outward particle flux is "Medium > 0", and moderate reversed particle flux is "Medium < 0".

profile tuning of the transport coefficients [27], and assuming that these coefficients can be extrapolated to the whole poloidal plane only taking into account the radial dependence. This is the standard procedure used in 2D transport modeling of plasma wall interaction. The patterns of strong outward particle flux are located on the one hand at the core to SOL interface, in both the high and low field regions, and on the other hand in the SOL region of the divertor for both the high and low field side legs. Finally, the region with moderate particle outflux exhibits poloidal symmetry around the core plasma and along the divertor legs. This very homogeneous transport pattern did not provide agreement with the experimentally estimated transport properties based on the measurement of the SOL widths. Furthermore, such symmetric transport pattern is known to disagree with experimental evidence that suggests ballooned transport [29, 30, 31]. Another simulation was performed with a different choice of transport coefficients, tuning the radial profile to fit the midplane profiles [27] and then extrapolating these to the poloidal plane by enforcing strongly ballooned transport, Figure 3, panel (b). A modest improvement when comparing to the TCV data is reported [75].

Let us now analyze TOKAM3X and SolEdge2D-EIRENE with the κ -only turbulent transport model simulations, Figure 4. The first observation that can be made is the clear imbalance between the high field side region with no significant turbulent transport and the low field side region where cross-field transport is localized. The transport

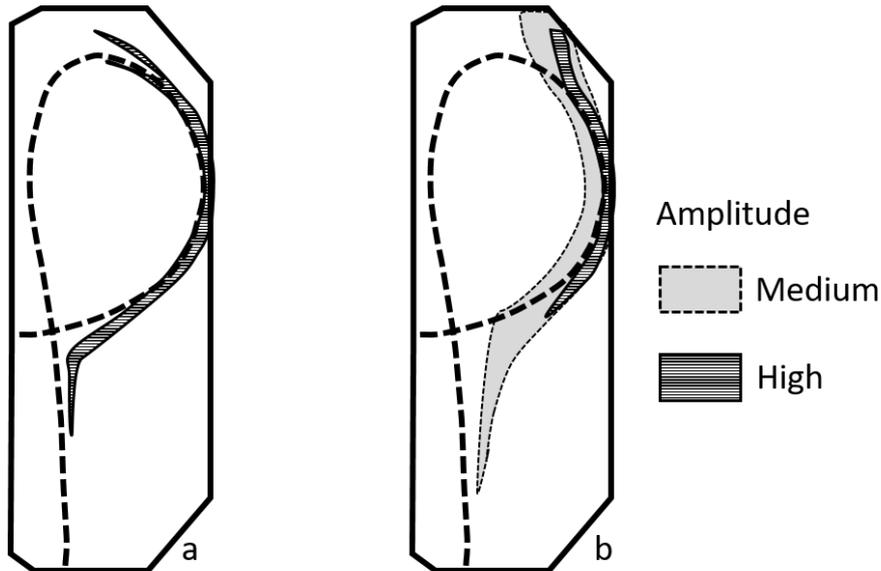


Figure 4. Sketch of the 2D transport pattern in TCX in the large X-point height configuration. Panel *a*: effective particle diffusivity given by TOKAM3X, sketch of the results published in Ref.[75]. Panel *b*: Particle turbulent diffusion coefficient D_n self-consistently determined in SolEdge2D-EIRENE simulation using the κ -only model, sketch of the results published in Ref.[76]. Regions with largest particle diffusion coefficient are labeled "high", those with still significant particle diffusion are labeled with "medium".

coefficient is observed to increase from the inner core boundary towards the separatrix, following a universal trend of the fluctuation profile measurements [77, 78, 79], recovered in some gyro-kinetic simulations [80]. Regarding the ballooning aspect, one finds that the poloidal extent is typically $\pm 80^\circ$, therefore broader than the $\pm 45^\circ$ reported for the Tore Supra limiter experiment [30, 31, 81]. One can notice also that the ballooned aspect is superimposed to a near constant D_n pattern extending poloidally in the low field side SOL and following approximately the separatrix. Comparing panels (a) and (b) indicates that qualitative agreement is achieved. The ballooned feature agrees with experimental observations Refs.[29, 30] and the localization along the low field side divertor leg as reported with visible imaging of turbulent filaments as in the MAST tokamak [82].

The particular choice made for the growth rate (19) therefore generates a ballooned structure of turbulent transport that exhibits key similarities with micro-turbulence simulations [83, 80, 75]. It clearly indicates that using such a growth rate of the form (19) is quite effective in providing a means to capture self-consistently a major property of turbulent transport throughout the plasma volume [29, 30, 81].

2.4. Diffusive transport driven by the local dynamics: closure constraint for D_ω

To address the closure of the κ - ε model with respect to D_ω and V , we consider the local evolution equation for κ and ε (17):

$$\partial_t \kappa = \gamma_\kappa \kappa - \frac{1}{D_\omega} \kappa^2 - \varepsilon \quad (20a)$$

$$\partial_t \varepsilon = \gamma_\varepsilon \varepsilon - \frac{V}{\kappa^{3/2}} \varepsilon^2 \quad (20b)$$

The control parameter of the quadratic term of the equation for κ , D_ω in (20a), ensures that κ has no fixed point at infinity and remains therefore bounded. Such a property is most important in predator-preys models when all fixed points are lost so that the system must then enter a limit cycle. The parameter D_ω is a free parameter, with dimension $m^2 s^{-1}$ akin to a diffusion coefficient. It can also be written as $D_\omega = \kappa_\omega / \Delta\omega$. The coefficient D_ω is split into the frequency width $\Delta\omega$ of the turbulent energy spectrum and a characteristic value of the latter κ_ω , providing a connection to published predator prey models [58]. As shown in Section 1, the steady state solution is obtained as the solution of a second order equation. It can be recast into an equation for the diffusion coefficient $D = \kappa^2 / \varepsilon$:

$$D^2 + DD_\omega - D_V D_\omega = 0 \quad (21a)$$

$$D_V = \frac{\gamma_\kappa V^2}{\gamma_\varepsilon} \quad (21b)$$

Note that for D_V to be positive one assumes here that $\gamma_\kappa > 0$. this is the condition to have solutions different from $D = 0$, therefore one positive solution determined by:

$$D = -\frac{1}{2}D_\omega + \frac{1}{2}\sqrt{D_\omega^2 + 4D_V D_\omega} \quad (21c)$$

Depending on V , one finds therefore two regimes, the weak turbulence regime for $D_V \ll D_\omega$, such that the expansion of the solution for D in (21c) yields $D \approx D_V$, which scales like V^2 , and a Bohm strong turbulence regime such that $D \approx \sqrt{D_V D_\omega}$ and is therefore linear in V [84]. The ratio D_V / D_ω , which can be interpreted as a Kubo number [85, 86], governs this transition. As a possible closure of the system, we enforce the diffusion coefficient D to be in the weak turbulence regime. In such a framework, we therefore consider that the transition to the Bohm regime for D will occur for much larger values of D than required to match the experiments. Consequently, we choose the parameter D_ω such that $D_\omega \gg D_n$, where D_n is the typical diffusion coefficient consistent with the SOL width. The parameter D_ω then only plays a role during transients with large κ amplitude. In the simulations D_ω is defined in terms of $\kappa_{max} = 10^{10}$, $D_\omega = \kappa_{max} / \gamma_\kappa$, where γ_κ is a characteristic value of the growth rate. For the sake of simplicity, D_ω is also assumed to be constant in the whole simulation volume. In this work we have therefore set the value of the parameter D_ω so that it has a minor role on the steady-state behavior of the κ - ε system.

2.5. SOL width driven by the local dynamics: closure constraint for V

Given the choice made for the growth rate $\gamma_\kappa = \gamma_\varepsilon = \gamma_I$ (19), the local fixed point yields $\kappa = V^2$ and consequently $D \approx D_V \approx V^2/\gamma_I$. The choice for the closure of the control parameter V is a key element of the model since it characterizes the magnitude of the turbulence energy and consequently that of turbulent transport. We have chosen here to define V with the aim of improving the predictive capability of simulations of plasma-wall interaction, in particular divertor operation. The control parameter V is then chosen such that the local evolution properties allow one to recover the SOL width λ_{SOL} as determined by the L-mode empirical scaling [87]. Since the scaling law is consistent with data from several devices, the choice of V should be appropriate to describe this class of devices with no further free parameter tuned to ensure the match. For the simulations, we have therefore enforced the relationship:

$$D_n = C_\nu D \approx \frac{\lambda_{SOL}^2}{\tau_{\parallel}} \quad (22a)$$

where $\tau_{\parallel} = L_{\parallel}/c_s$ is the parallel connection characteristic time, c_s the sound velocity and $L_{\parallel} = qR$ the connection length. The recent scaling laws for the SOL width are based on a proportionality to the so-called poloidal Larmor radius $q\rho_0$ [87, 88, 89]. This scaling property is readily recovered with simple arguments. First the SOL width is a length and consequently must scale as a length. Second, it is understood that micro-turbulence determines the SOL width so that the characteristic length associated to micro-turbulence, the Larmor radius ρ_0 , is the reference length of interest, consequently $\lambda_{SOL}/a \propto \rho_*$ where $\rho_* = \rho_0/a$. However, dimensionless parameters can still play a role. Using the long standing empirical dependence on the plasma current [90], hence on the poloidal magnetic field rather than the toroidal magnetic field, one can then readily conjecture:

$$\frac{\lambda_{SOL}}{a} = \alpha_s q \rho_* \quad (22b)$$

where α_s is a matching scalar parameter. Alternatively, as shown in Ref.[59] and sketched here, one can consider the quasi-linear framework for electrostatic turbulence - although questionable given the large amplitude fluctuations in the SOL- with $\lambda_{SOL}/a = (D_{ql}\tau_{\parallel}/a^2)^{1/2}$. Then for $D_{ql} \propto (\rho_* v_0)^2 \tau_{\parallel}$, one obtains $\lambda_{SOL}/a \propto q\rho_* A$, where $A = R/a$ is the aspect ratio. Such a result holds provided one assumes that the characteristic time τ_{ql} (12c) is identified to be proportional to τ_{\parallel} . The question of the SOL width scaling has been a long standing issue and the matter of considerable work as for the ITER design [91]. Only recently [88] has a particular empirical scaling law been considered to be generic. However, one could argue that the scaling law is assumed and that the significant error bars on the exponents [87] accommodate for the experimental dispersion. Note that data base used for the empirical analysis [87] does not account for aspect ratio A variation. The dependence on such a parameter would be useful to discriminate various models. The results discussed here are also tightly related to

choosing τ_{\parallel} as characteristic time. Given the closure (18) yielding $D_n = C_{\nu}D$ and that introduced here (22a), we then obtain $D \approx D_{\nu}$:

$$D = \frac{\lambda_{SOL}^2}{\tau_{\parallel}^2} \frac{\tau_{\parallel}}{C_{\nu}} = \frac{\lambda_{SOL}^2/a^2}{\tau_{\parallel}^2/a^2} \frac{\tau_{\parallel}}{C_{\nu}} = \rho_*^2 c_s^2 \tau_{\parallel} \frac{\alpha_s^2/A^2}{C_{\nu}} \quad (22c)$$

One can then determine the control parameter V where the combination of proportionality factors defines the factor $V_0 = \alpha_s/\sqrt{C_{\nu}}$.

$$V = c_s \rho_* \frac{V_0}{A} \sqrt{\frac{\tau_{\parallel} \gamma_{\varepsilon}^2}{\gamma_{\kappa}}} \quad (22d)$$

The typical value we thus obtain for the parameter V is therefore $\rho_* c_s$, the order of magnitude of the drift velocities. For high-confinement and low-gas-puff plasma regime, the H-mode scaling law leads to $\lambda_{SOL}/a \sim 2q\rho_*$ [88], hence $\alpha_s \sim 2$. In L-mode, the SOL width is typically 2 – 3 times larger with the same scaling [87], $\alpha_s \approx 4 - 6$.

Since the growth rates γ_{κ} and γ_{ε} are defined as local parameters, one readily finds that V (22d) is also a local parameter. The latter appears as the local effective velocity of micro-turbulent transport. For magnetized plasmas, the magnitude of these velocity fluctuations is typically $c_s \rho_*$, which is the characteristic amplitude of the $E \times B$ electric drift velocity. The transport cross-field velocity will depend on this drift velocity as well as some form of repetition rate of the transport events. Indeed the latter can occur in bursts followed by short quiescent phases [92]. The effective velocity V must account for this behavior, typically multiplying the drift velocity by the ratio of the burst duration divided by the time delay between two bursts of turbulence. We therefore use *global* features such as the interchange growth rate and the empirical scaling of the SOL width to constrain the *local* free parameters of the κ - ε model. The latter introduces new possibilities for transport self-organization, but these do not enforce that the global properties of the simulations will exhibit the empirical SOL width scaling. To obtain the expression of V , we have used the characteristic time scale $\tau_{\parallel} = qAa/c_s$. In fact two time scales appear in the model, τ_{\parallel} and the linear time scale associated to the growth rate of the interchange instability, namely $1/\gamma_I$ (19).

$$\gamma_I \tau_{\parallel} = q \sqrt{R^2 \frac{\nabla p \cdot \nabla B_T}{p B_T}} \propto q A^{1/2} \quad (23a)$$

We assume here that $\nabla B \approx B/R$ while that $\nabla p \approx p/a$, this scaling property is commonly considered in linear approximations. To take this scaling into account while retaining the dependence on the departure from equilibrium Θ we then define:

$$\gamma_I \tau_{\parallel} = q A^{1/2} G(\Theta) \quad ; \quad \Theta = \frac{a \nabla p}{p} \cdot \frac{R \nabla B_T}{B_T} \quad (23b)$$

The function $G(\Theta)$ is null for $\Theta < 0$ and otherwise equal to $\sqrt{\Theta}$. One then obtains for V_{τ} the value of V for $\tau/\tau_{\parallel} = 1$:

$$V_{\tau} = c_s \rho_* V_0 \frac{q^{1/2}}{A^{3/4}} \sqrt{G(\Theta)} \quad (24a)$$

Given a characteristic time τ and the velocity amplitude V one can define the length scales $\lambda_b = V\tau$ for ballistic transport and $\lambda_d = \sqrt{D\tau}$ for diffusion. In the first part of this Section, the time scale τ is chosen to be τ_{\parallel} , leading to different ballistic and diffusive expressions of the SOL width, when setting $\gamma_{\kappa} = \gamma_{\varepsilon} = \gamma_I$ one obtains $\lambda_b/\lambda_d \approx (\tau\gamma_I)$. However, when retaining $\tau = 1/\gamma_I$ the ballistic and diffusive length scales defined above are identical. With the latter assumption, one obtains V_{γ} :

$$V_{\gamma} = c_s \rho_* V_0 \frac{q}{A^{1/2}} G(\Theta) \quad (24b)$$

For both cases, one has $\kappa = \gamma_{\kappa}^2 V^2 / \gamma_{\varepsilon}^2$, $\varepsilon = \gamma_{\kappa} \kappa$ and $D = \gamma_{\kappa} V^2 / \gamma_{\varepsilon}^2$, so that one has:

$$D_{\tau} = ac_s V_0^2 q \rho_*^2 A^{-1} \quad ; \quad \tau/\tau_{\parallel} = 1 \quad (25a)$$

$$D_{\gamma} = ac_s V_0^2 q^2 \rho_*^2 A^{-1/2} G(\Theta) \quad ; \quad \tau\gamma_I = 1 \quad (25b)$$

The choice of the characteristic time scale has an impact of the final result. To account for this fact, we introduce three undetermined exponents ϵ_{ρ} , ϵ_q and ϵ_A in the definition of D .

$$D \propto ac_s (\rho_*^2)^{\epsilon_{\rho}} q^{\epsilon_q} A^{\epsilon_A} \quad (26)$$

For $\tau\gamma_I = 1$, $\epsilon_q = 2$, $\epsilon_A = -0.5$ while for $\tau = \tau_{\parallel}$ $\epsilon_q = 1$, $\epsilon_A = -1$. The exponent of ρ_*^2 allows one taking into account a possible ρ_* dependence of the characteristic time scale. A gyroBohm scaling, such that τ does not depend on ρ_* , is recovered with $\epsilon_{\rho} = 1$, and a Bohm scaling with τ depending on ρ_* is obtained with $\epsilon_{\rho} = 1/2$. Expression (26) provides a more general framework than has been used in the simulation work presented in this paper. It will be used when comparing empirical scaling laws and that derived from the κ - ε model for the energy confinement time and SOL width. Regarding the simulations, with both the 1D model and SolEdge2D, we have used the closure V_{τ} , slightly modified. Indeed, when $G(\Theta) \rightarrow 0$, for instance on the high field side, both fixed points values of κ and ε tend to zero. This occurrence is difficult to handle numerically and the expression for the diffusion coefficient is modified to be robust against numerical overflow with $D = \kappa^2 / (\varepsilon + \varepsilon_f)$. The parameter ε_f is small and is introduced to avoid any spurious divergence when $\varepsilon \rightarrow 0$. As a consequence, at the limit $G(\Theta) \rightarrow 0$, one obtains that $D \rightarrow 0$ rather than the limit given by (25a). Recovering the latter would be numerically demanding as well as being nonphysical. Further investigation of this point has been achieved and is to be published [74].

2.6. Linear analysis of the κ - ε model

When investigating dispersion equations for plasma instabilities one obtains most often a growth rate that exhibits a threshold. This is a key property of self-organization that is not taken into account by the chosen form of the growth rate (19) for the present κ - ε model. By analyzing the linearized fluctuation equations we show that the transport

terms of the evolution equations (3b) generate a threshold for the effective growth rate. Let us consider the linearized κ - ε model in Fourier space:

$$\partial_t \widehat{\kappa} + ik_{\parallel} u_{\parallel} \widehat{\kappa} + k_{\perp}^2 D_{\kappa*} \widehat{\kappa} = \gamma_{\kappa} \widehat{\kappa} - \frac{2}{D_{\omega}} \kappa_* \widehat{\kappa} - \widehat{\varepsilon} \quad (27a)$$

$$\partial_t \widehat{\varepsilon} + ik_{\parallel} u_{\parallel} \widehat{\varepsilon} + k_{\perp}^2 D_{\varepsilon*} \widehat{\varepsilon} = \gamma_{\varepsilon} \widehat{\varepsilon} - \frac{2V}{\kappa_*^{3/2}} \varepsilon_* \widehat{\varepsilon} + \frac{3V}{\kappa_*^{5/2}} \varepsilon_*^2 \widehat{\kappa} \quad (27b)$$

where κ_* , ε_* are the steady state solution homogeneous in space, hence solutions of the local system (20) and where $\widehat{\kappa}$ and $\widehat{\varepsilon}$ are the Fourier modes with wave vectors k_{\parallel} and k_{\perp} . The parallel velocities are chosen to be constants and the diffusion coefficients $D_{\kappa*}$ and $D_{\varepsilon*}$ are computed with the steady state solution κ_* , ε_* . The system (27) describes the evolution of the system disturbed from the steady state solution by a perturbation varying in space. The dispersion relation is then given by setting the determinant of the linear system to zero and takes the generic form:

$$(\gamma - A)(\gamma - B) + C = 0 \quad (28a)$$

The coefficients A , B , C of this dispersion relation are:

$$A = \gamma_{\kappa} - k_{\perp}^2 D_{\kappa*} - \frac{2}{D_{\omega}} \kappa_* - ik_{\parallel} u_{\parallel} \quad (28b)$$

$$B = \gamma_{\varepsilon} - k_{\perp}^2 D_{\varepsilon*} - \frac{2V}{\kappa_*^{3/2}} \varepsilon_* - ik_{\parallel} u_{\parallel} \quad (28c)$$

$$C = \frac{3V}{\kappa_*^{5/2}} \varepsilon_*^2 \quad (28d)$$

Stabilizing effects are governed by the diffusion coefficients of the form $k_{\perp}^2 D_*$ and by parallel transport, the latter depending on the physics controlling the Doppler shift $k_{\parallel} u_{\parallel}$. The contribution of the transport terms thus generate damping processes that inhibit the drive terms and consequently govern the occurrence of threshold effects. One then recovers effectively a more complex form of the growth rate, including in particular a threshold in pressure gradient as well as damping processes governed by parallel transport. These are usually taken into account when computing γ_I as in Refs.[34, 39, 68]. However, since the physics of parallel transport and diffusive cross-field transport appear independently in the evolution equations of κ - ε (17), the present approach consists of using the growth rate proportional to γ_I (19) and let the physics included in the κ - ε system (17) modify this drive.

The closure based on the SOL width involves the choice of the characteristic time but also that of the dominant transport process, either ballistic with characteristic velocity V_b or diffusive with diffusion coefficient D . The standard approximation for transport is to consider a diffusive process such that the flux depends on a gradient and consequently on the departure from thermodynamic equilibrium. Introducing ballistic transport, typically of the form $V_b n$ for density yields transport even without departure from thermodynamic equilibrium, which simply makes no sense unless V_b itself depends on

a gradient. In the present case, both expressions of V_τ (24a) and V_γ (24b) depend on the departure from equilibrium Θ (23b). It would therefore be possible to add to the diffusive transport an outward ballistic transport proportional to $\kappa^{1/2}$ since $\kappa \neq 0$ is governed by a departure from equilibrium, either directly via the growth rate of the form 19 or via spreading from a region with positive growth rate. In the present work, for the sake of simplicity, we make the choice of only retaining diffusive transport as done in neutral fluids. Still, the non-linearity of the diffusion coefficient can generate transport properties that are quite similar to ballistic transport. Indeed, the diffusive transport of any field F with diffusion coefficient of the form $D = \kappa^2/\varepsilon$ yields a divergence of the flux of the form:

$$-\nabla_\perp(D\nabla_\perp F) = -D\nabla_\perp^2 F - D\nabla_\perp F \left(2\frac{\nabla_\perp \kappa}{\kappa} - \frac{\nabla_\perp \varepsilon}{\varepsilon} \right) \quad (29a)$$

The divergence is simplified by considering here a slab geometry. Such a structure is comparable to that induced by convective transport, hence driven by a flux of the form $-D_{pinch}\nabla_\perp F + V_{pinch}F$, where V_{pinch} must stand for the coupling to another field and be proportional to the gradient of that other field. The divergence of this flux is then:

$$\begin{aligned} \nabla_\perp(-D_{pinch}\nabla_\perp F + V_{pinch}F) &= -D_{pinch}\nabla_\perp^2 F - D_{pinch}\nabla_\perp F \left(\frac{\nabla_\perp D_{pinch}}{D_{pinch}} - \frac{V_{pinch}}{D_{pinch}} \right) \\ &\quad + (\nabla_\perp V_{pinch})F \end{aligned} \quad (29b)$$

Identifying the latter form of the flux divergence with expression (29a) for constant D_{pinch} and V_{pinch} , one obtains $V_{pinch} = D(\nabla_\perp \text{Log}(\varepsilon) - 2\nabla_\perp \text{Log}(\kappa))$. The structure of these two transport models departs when the variation of V_{pinch} , $\nabla_\perp V_{pinch}$ contributes significantly to the transport. This seems to be a rather particular transport property. But for this specific case, the fact that κ and ε can vary in space generates transport features that are reminiscent of convection although the mathematical structure of the transverse transport term is diffusive. The non-linear dependence of D in κ and ε can therefore generate complex transport properties that depart significantly from the standard case with constant diffusion parameter, such as spreading [93]. Given these properties, the reduced κ - ε model of cross-field transport can be relevant to model slow transients and not only steady states. This is particularly important for a non-linear system, such as that governing plasma-wall interaction, since one cannot assume that a steady state does exist. Indeed, clear oscillatory behavior has been reported [94].

3. Confrontation of κ - ε transport with scaling laws

We first test the model in a 1D transport model. The results can then be compared to the available empirical scaling laws characterizing global properties such as the energy confinement time and the SOL width. One can also test means to extend the capability of the model towards self-consistent generation of transport barriers Section 3.4.

3.1. The 1D κ - ε transport model

The 1D model is obtained by averaging the transport equations in the poloidal and toroidal directions. Transport is taken into account by diffusion that replace all convective transport processes at the microscopic level. The diffusion coefficients are chosen as discussed in Section 2.1 equation (18). The plasma transport equations for the particle density, and the density of electron and ion thermal energy are presented in Appendix A. In these equations a mask function defines the SOL region for $r \geq a$ where parallel losses to the wall apply. The plasma equations are completed by a vorticity equation (A.1b). It will be used when addressing the impact of velocity shear on the transport properties in Section 3.4. A simplified transport model for neutrals is also introduced so that the particle source by ionization, and in particular its location, changes consistently with the plasma parameters. A feed-back controlled gas injection Φ_{N0} is set to ensure that the separatrix density is fixed, equal to $1 \cdot 10^{19} \text{ m}^{-3}$. The cross-field diffusion coefficients of plasma transport are determined by the κ - ε coupled equations (17), restricted to the 1-D radial direction (A.1d). A core energy source with given radial profile is used. The scan in additional heating power is performed by varying the source amplitude (A.2b).

3.2. Impact of a parameters scan on the SOL width

The SOL width estimated by the numerical model is compared to the empirical scaling law for the heat flux width λ_q given by ref.[87] in *mm* for L-mode discharges:

$$\lambda_q = (1.44 \pm 0.67) B_T^{-0.8 \pm 0.32} q^{1.4 \pm 0.67} P_{in}^{0.22 \pm 0.1} R^{-0.03 \pm 0.28} \quad (30a)$$

As shown in Appendix B, this scaling law is not dimensionally correct since the adimensional ratio λ_q/a still depends on B_T as well as on the dimensionless parameters q , A , β , ν_* and ρ_* .

$$\frac{\lambda_q}{a} \propto q^{2.15} A^{0.03} \beta^{0.46} \nu_*^{-0.17} \rho_*^{0.85} B_T^{0.156} \quad (30b)$$

Within the error bars of (30a), the only modification of a single exponent that allows one recovering a dimensionally correct expression is to change the exponent of the power law on B_T from -0.8 to $-0.8 - 0.156 = -0.956$ so that the same expression (30b) is obtained but with an exponent for B_T equal to zero. In order to determine the dependence of λ_q/a on the dimensionless parameters, we use the dimensionless scaling law for the energy confinement time *ITER96-th* [91]. It is to be remarked that also for this empirical scaling law, a correction on the exponent of R is made to achieve a dimensionally correct expression [91], see also Appendix B. In both cases, the modification of the quality of the scaling law by the dimensionality constraint, the so-called Kadomtsev constraint [95, 96, 97], is lacking. Finally, it is to be underlined that the assumption $\lambda_{SOL}/a \propto q\rho_*$ used to define the free parameter V , Section 2.5 (22d) is not recovered here since one obtains typically $\lambda_q/a \propto q^{2.15} \rho_*^{0.85}$. There seems to be a consistency issue between the

assumed dependence $\lambda_q/a \propto q\rho_*$ as claimed in the literature and the effective scaling law and that stemming from the proposed scaling law.

Since the decay rate of the energy flux in the SOL region depends on the parallel heat conductivity, which is not taken into account by the 1D model, we determine here the e-folding length of the pressure profile λ_p and use it as a proxy for λ_q . The fall-off region of the pressure profile used to determine λ_p is restricted to the vicinity of the separatrix, typically the first 20% of the SOL width. Based on the oversimplified use of e-folding lengths of density and temperature, one can predict that $\lambda_q/\lambda_p \approx 0.75\ddagger$, this ratio being a constant depending on the sheath transmission factor and respective heat and particle cross-field transport. The SOL width determined by this method could overestimate the actual width but should exhibit the same parameter dependence.

In a first series of simulations analyzed here, we perform a scan of the input power $P_{in} \in [0.1, 0.5, 1, 2, 3, 4, 6]$ MW. The lowest values of P_{in} of 0.1 and 0.5 MW are not realistic but useful when analyzing the results in terms of scaling laws. The particle source is adjusted to impose the same density at the separatrix $n_a = 10^{19}m^{-3}$. One can then expect that the neutral penetration into the plasma will be roughly constant so that the ionization source will have the same shape but varying amplitude to match the particle outflux. Experimentally this would correspond to a feedback process on a density measurement at the midplane separatrix, a control scenario particularly suitable to investigate divertor physics [98]. With such a scenario for the power scan one readily expects an increase of the plasma thermal energies T_e and T_i with P_{in} as well as an increased SOL width given the scaling law (30a). Based on a straightforward analysis [99], one can show that the ratio between core and separatrix density n_0/n_a is expected to be $n_0/n_a = 1 + \lambda_I/\lambda_{SOL}$. The neutral ionization mean free path λ_I determines the radial distance between the particle source and the separatrix. One thus finds that the source is all the more effective that it is closest to the plasma center. This effect is balanced by particle transport characterized by λ_{SOL} . The main change when scanning the power is then the impact on the transport features characterized by λ_{SOL} . This effect drives a reduction of the core density with increasing power. These trends are recovered in the simulations, Figure 5. On the top-left figure labeled (a), are plotted the outer density profiles, these being characterized by flat core profiles. As the power is increased the core density decreases while the density profile at the separatrix becomes flatter. The ratio n_0/n_a is observed to decrease - n_a the separatrix density is feedback controlled to be constant- when increasing P_{in} , Figure 6 left hand side. However, one can observe that the density decrease is less pronounced for $P_{in} \geq 1MW$ compared to the very low power cases. Conversely, the thermal energies, Figure 5 top right labeled (b), increase with the injected power. Analyzing the core T_0 and separatrix T_a thermal energy, one finds that the temperature profile scales with the injected power, $T_a \propto P_{in}^{0.537}$ and $T_0 \propto P_{in}^{0.513}$, 6

‡ This value is estimated using an e-folding length for both density λ_n and temperature λ_T , and given equal ion and electron temperatures. Assuming that the profile of the heat flux is governed by the sheath conditions, one then obtains $\lambda_q/\lambda_p = (1 + \lambda_T/\lambda_n)/(\frac{3}{2} + \lambda_T/\lambda_n)$ equal to $\frac{2}{3}$ for $\lambda_T/\lambda_n \rightarrow 0$ and increasing to 1 for $\lambda_T/\lambda_n \rightarrow +\infty$. The quoted value $\lambda_q/\lambda_p = 0.75$ is obtained for $\lambda_T/\lambda_n \rightarrow 0.5$.

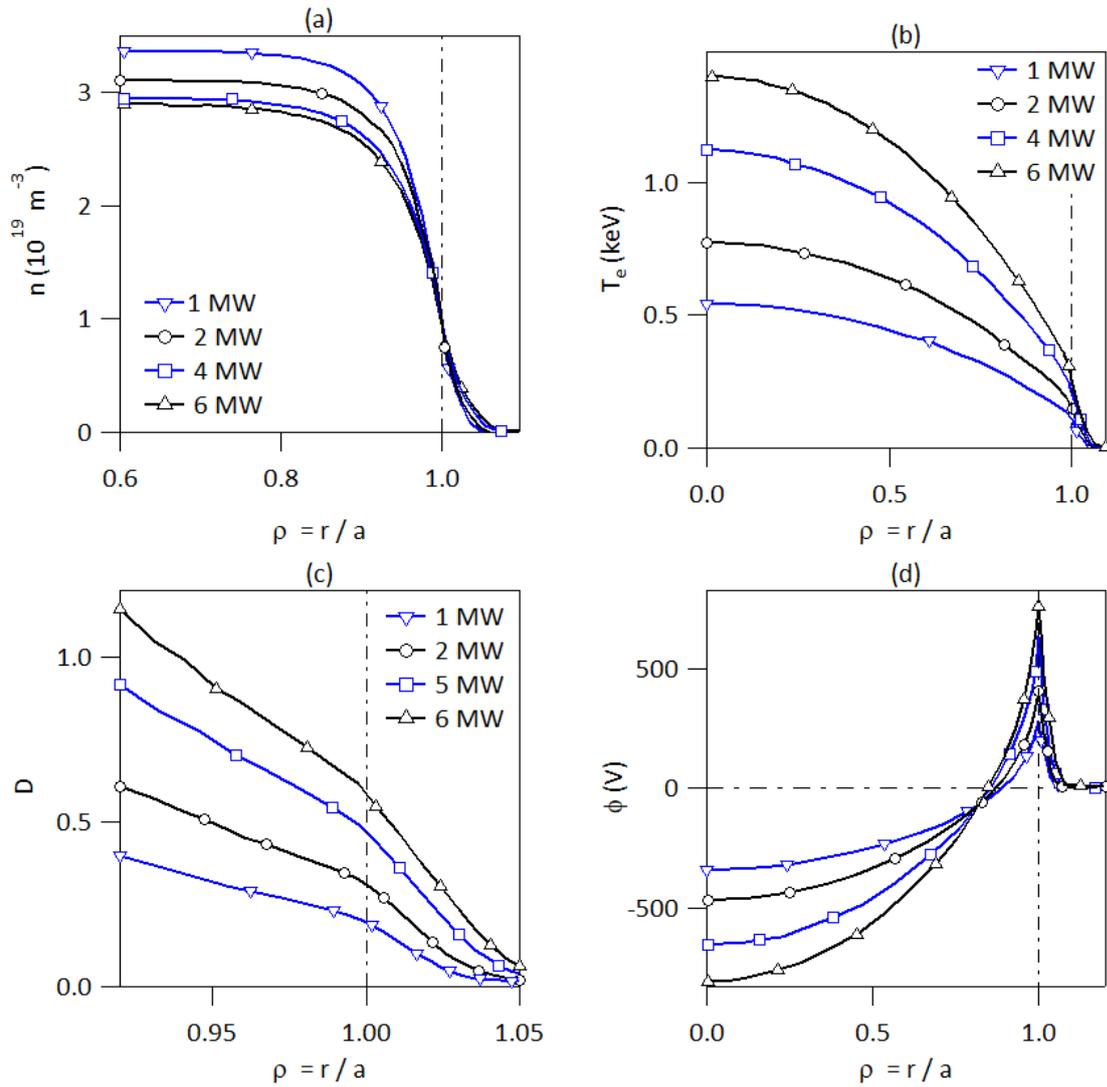


Figure 5. Radial profiles of plasma quantities when increasing the input power with feedback control on the separatrix density. (a) density. (b) Electron temperature. (c) Effective diffusion coefficient estimated from κ and ε . (d) Electric potential.

right hand side. The overall effect on the plasma pressure $p = nT$ is an increase. In the core, Figure 6 left hand side, the pressure increase p_0 does not exhibit a power law because of the particular response of the core density. At the separatrix, where the density is maintained constant by the chosen feedback, the pressure is governed by the behavior of the thermal energy, and exhibits the same power law with respect to the injected power. The pressure gradient at the separatrix drives the turbulent energy κ as well as the turbulent energy transfer rate ε , these determine the plasma transport and consequently the pressure gradient at prescribed fluxes: feedback controlled for the particles and externally controlled for the energy. The particle radial diffusion D_n governed by the fluctuating convection process is a key element of this non-linear loop: see Figure 5 bottom left labeled (c) where the profiles of D_n are plotted for different powers P_{in} in

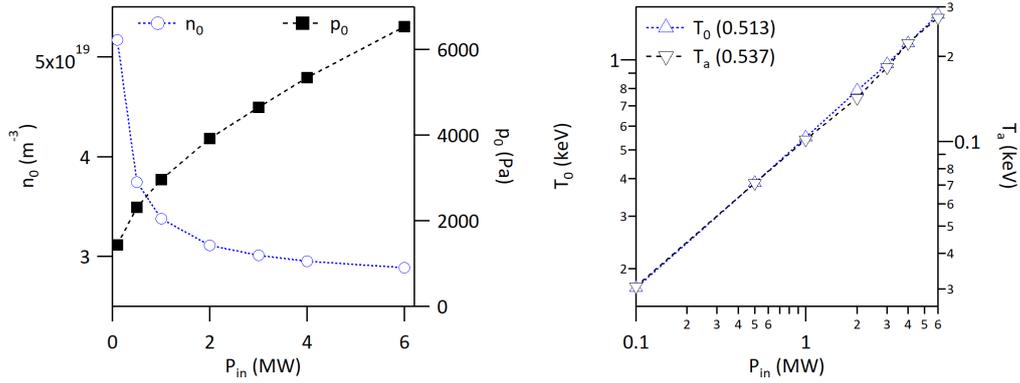


Figure 6. Left hand side: Dependence of the core density n_0 and of the core pressure p_0 on the input power P_{in} . Right hand side: scaling of the core T_0 and separatrix T_a thermal energy in terms of the input power. The values in bracket are the scaling exponents.

the vicinity of the separatrix. The profiles are similar in the region $r \leq a$, and exhibit differences in the SOL $r > a$ and in particular in the far SOL region at lowest power in regions where the plasma is weakly ionizing. Considering the values at the separatrix,

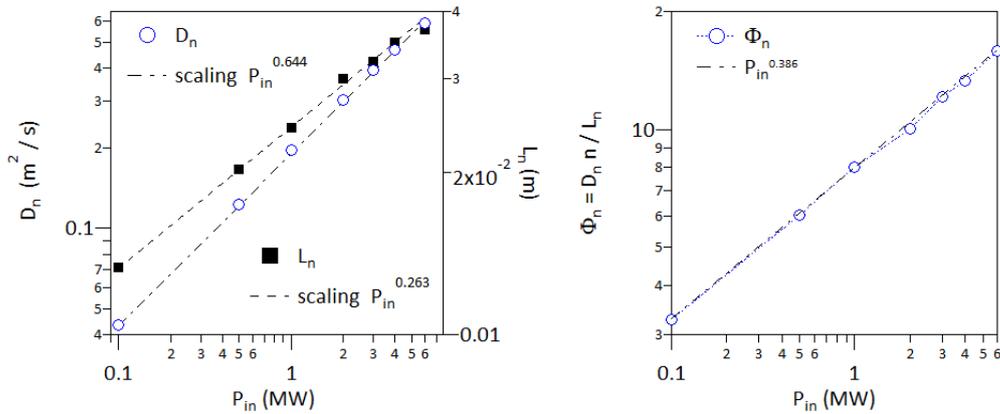


Figure 7. At the separatrix, power law dependence on the input power P_{in} , Left hand side: particle diffusion coefficient D_n and density gradient length $L_n = (\nabla_r n/n)^{-1}$, Right hand side: neutral particle flux Φ_n .

one finds that D_n exhibits a power law dependence on the injected power, $D_n \propto P_{in}^{0.644}$, Figure 7 left hand side. Furthermore, from the density profiles Figure 5 top-left, one can determine the density gradient length $L_n = (\nabla_r n/n)^{-1}$. The latter is also observed to scale with the injected power, $L_n \propto P_{in}^{0.263}$, Figure 7 left hand side. Assuming that this scaling is consistent with the choice made to determine the parameter V (22d), one expects $L_n/a \propto q\rho_*$ and thus $L_n \propto T_a^{1/2}$, yielding the power law exponent 0.269 in good agreement with that obtained for L_n . One can also compare the scaling law for L_n to that observed experimentally as described by the scaling law of λ_q (30a). Balancing the cross-field diffusive flux and the parallel losses, one expects the exponent of L_n to

be 0.19, half of the exponent of D_n minus one fourth of the exponent of the thermal energy T_a . Both exponents stand within the error bars of the experimental scaling law (30a). Their difference stems from the particle source in the SOL that must be taken into account for the steady state particle flux balance together with the parallel and cross-field fluxes. One can expect this particle source to exhibit another power law dependence on P_{in} than the two flux divergence terms. One thus finds that the SOL behavior is governed by the balance of three terms that are characterized by different underlying scaling laws. The overall behavior can only exhibit a power law, governed by a single exponent, on a restricted range of P_{in} . The amplitude of the ionization source term can be addressed by analyzing the cross-field particle flux at the separatrix that balances in steady state neutral influx. The flux of neutrals that are ionized in the edge plasma is therefore determined by $\Phi_n = D_n n / L_n$ at the separatrix, Figure 7 right hand side. This flux, and consequently the particle source, is governed by the feedback loop that maintains the separatrix density $n_a = 10^{19} \text{ m}^{-3}$. Owing to the enhanced particle transport with the injected power, the neutral particle influx must increase with the latter and indeed one finds $\Phi_n \propto P_{in}^{0.386}$.

A last observation from the simulation is the plasma electric potential profiles shown on Figure 5 bottom-left labeled (d). One finds that a potential well develops in the core, typically governed by the ion temperature profile, while the SOL values are positive and slaved to the electron temperature in the SOL, the outer wall being grounded and defining the reference electric potential. As a consequence of this drive, a maximum $E \times B$ shear region is located in the vicinity of the separatrix where the electric field reverses.

Let us now address the scaling of λ_{SOL} determined by the pressure e-folding length λ_p and compare it to the experimental scaling law (30a). On Figures 8 and 9 are plotted the variation of λ_p from the present κ - ε 1D model -black curves open squares- and power law variations: the experimental scaling law -dash dot black curve, and a fit of the results of the 1D model, dashed blue curve. The gray-shaded area stands for the 95% confidence interval of the regression used to determine λ_p .

parameter	P_{in}	B_{pol}	R	B_T
κ - ε power law fit	0.105	-0.844	-1.06	0.277
Scaling law (30a)	0.22	-1.4	-1.43	0.6 (0.444)

Table 1. Exponents of the power law dependence on the engineering control parameter for the fit of the κ - ε simulations and the experimental scaling law. The second value of the scaling law coefficient for B_T is that proposed to recover a dimensionally correct experimental scaling law.

Table 1 summarizes the various results for the main parameter scans. One can notice that with the chosen closure, the κ - ε model exhibits the same trend as the experimental

§ One considers here $L_n^2 = D_n L_{||} / c_s$ and therefore $L_n \propto D_n^{1/2} T^{-1/4}$.

scaling law since all the power law exponents have the same sign. However, the magnitude of these exponents observed in the κ - ε simulations are systematically smaller. We first discuss the results for the power scan, Figure 8 left hand side. The values obtained with the simulations are typically a factor 3 larger than the values given by the scaling law. Furthermore, there is a factor two in the exponents. For this parameter, the match between the experimental scaling law and the κ - ε simulations is rather poor. The dependence of λ_q on the magnitude of the poloidal magnetic field, hence on the plasma current, is governed by that on q in (30a) when the other engineering parameters are kept constant, hence $\lambda_q \propto B_{pol}^{-1.4}$. In the experimental findings on a single machine this is the main parameter dependence as evidenced in [90, 100]. The latter trend, typically $\lambda_q \propto B_{pol}^{-0.5}$ reported in [90, 100], or the direct behavior stemming from $\lambda_q \propto q$, hence $\lambda_q \propto B_{pol}^{-1}$ is approximately recovered with the κ - ε simulations, Figure 8 right hand side. However, this exponents departs by nearly a factor 1.5 from that in (30a). The

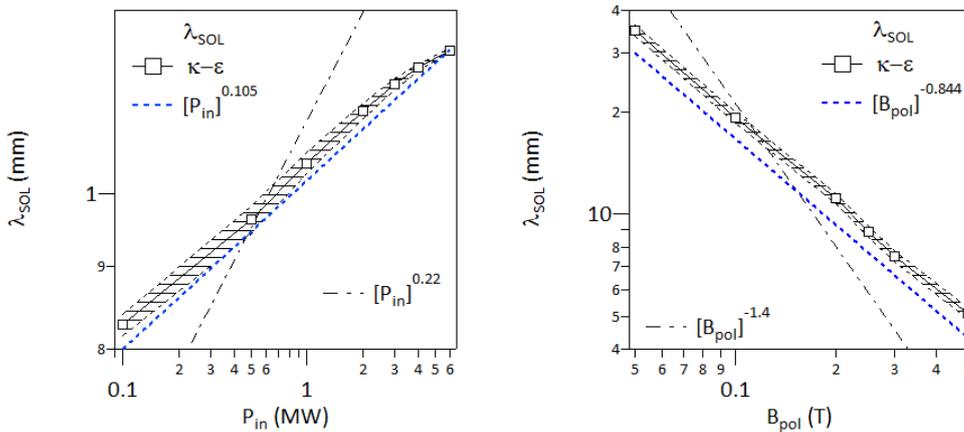


Figure 8. Power law dependence of the plasma pressure e-folding length, used as a proxy for the SOL width λ_{SOL} black open squares, approximate power law following these results dashed blue line, and power law governed by the experimental scaling law, black dashed-dot line. Left hand side scaling with $P_{in} \in [0.1 \text{ MW}, 6 \text{ MW}]$. Right hand side scaling with $B_{pol} \in [0.1 \text{ T}, 0.25 \text{ T}]$. The gray-shaded area stands for the 95% confidence interval of the regression used to determine λ_p .

dependence on R , keeping all other engineering parameters constant, hence minor radius, poloidal and toroidal magnetic field, density and input power, Figure 9 left hand side, is mainly governed in (30a) by the dependence of q on $1/R$, leading to $\lambda_q \propto R^{-1.43}$. Again the trend of the scaling law and the simulation agree, see Figure 9 left hand side, but the power law behavior is clearly different. Finally, when varying the toroidal magnetic field alone, Figure 9 right hand side, the κ - ε model yields a rather weak dependence while that of the scaling law is a factor two larger. The latter combines the explicit dependence on B_T as well as that embedded in the safety factor leading to the effective power law $B_T^{0.6}$. With the proposed correction such that the experimental scaling law fulfills the Kadomtsev constraint, the exponent for the B_T scaling is decreased to 0.444 reducing the mismatch with the κ - ε result.

When addressing the comparison between the κ - ε results and the scaling law (30a)

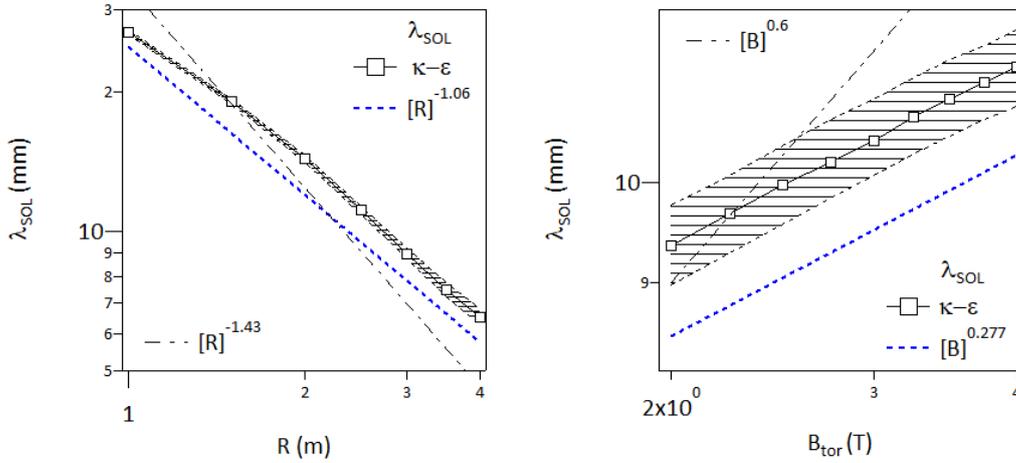


Figure 9. Power law dependence of the plasma pressure e-folding length, assumed as a proxy for the SOL width λ_{SOL} black open squares, approximate power law following these results dashed blue line, and power law governed by the experimental scaling law, black dashed-dot line. Left hand side scaling with $R \in [1 \text{ m}, 4 \text{ m}]$. Right hand side scaling with $B_T \in [2 \text{ T}, 3 \text{ T}]$. The gray-shaded area stands for the 95% confidence interval of the regression used to determine λ_p .

several issues are raised which can readily explain the observed differences. First, as we have just recalled, the reference expression is not dimensionally correct, second, when written in terms of the standard dimensionless parameters, one does not recover the assumed underlying form, and, third, the error bars on the coefficients of the power law are considerable. These issues can explain the mismatch, or when taking into account the error bars, enable finding some form of agreement despite significant differences in behavior and predictions. The κ - ε results are further discussed in the following Section considering the core energy confinement together with the SOL width.

3.3. Physics background of the scaling law for the SOL width and global energy confinement time

Considering the two definitions $\tau_E \approx a^2/D_{core}$ and $\lambda_q^2 = \tau_{\parallel} D_{SOL}$ one can readily determine the relationship between the energy confinement time and the SOL width, $\tau_E = \tau_{\parallel} (a/\lambda_q)^2 D_{SOL}/D_{core}$, therefore:

$$\tau_E \lambda_q^2 \propto \tau_{\parallel} a^2 \frac{D_{SOL}}{D_{core}} \quad (31a)$$

When assuming the same scaling properties for D_{SOL} and D_{core} , one can then estimate λ_q given the scaling of τ_E or vice-versa that of τ_E given that of λ_q .

$$\tau_E \propto \tau_{\parallel} \frac{a^2}{\lambda_q^2} \approx \frac{q^2 R}{\Omega_0 \lambda_q} \frac{R^2}{A^2 \lambda_q^2} = (qA^{-1})^2 (R\lambda_q^{-1})^3 \Omega_0^{-1} \quad (31b)$$

Such a connection between the global energy confinement time and the width of the boundary layer has been investigated experimentally [101]. With the approximation $\lambda_q \approx q\rho_0$ with $\rho_0 = c_s/\Omega_0$, Ω_0 being the reference ion Larmor gyration pulsation, one obtains the confinement time $\tau_{E,\lambda}^{(q\rho_0)}$:

$$\Omega_0 \tau_{E,\lambda}^{(q\rho_0)} \propto q^{-1} A \rho_*^{-3} \quad (31c)$$

Using the scaling of λ_q (30a) obtained for L-mode operation, one thus obtains a scaling law for τ_E^L

$$\tau_{E,\lambda}^L \propto q^{-2.2} A^{-2} R^{3.09} B^{1.4} P_{in}^{-0.66} \quad (32a)$$

Recalling the approximation made for the safety factor $1/q \approx A^2 I_p / (RBk)$, where I_p is the plasma current and k the elongation, see Appendix B, the scaling for τ_E derived from that of the SOL width is:

$$\tau_{E,\lambda}^L \propto I_p^{2.2} B^{-0.8} P_{in}^{-0.66} R^{0.89} A^{2.2} k^{-2.2} \quad (32b)$$

The latter expression is to be compared to a scaling law for the energy confinement time, such as the *ITER96-th* scaling law for L-mode [91]:

$$\tau_{E,th}^L = 0.023 I_p^{0.96} B^{0.03} P_{in}^{-0.73} R^{1.83} A^{0.06} k^{0.64} n^{0.4} M^{0.2} \quad (33a)$$

where τ_E is obtained in [s], for I_p , the plasma current in [MA], B_T the toroidal magnetic field in [T], P the loss power in [MW], n the line-averaged density in [$10^{19}m^{-3}$], M the average ion mass in [AMU], R the major radius in [m] and finally A and k the aspect ratio and elongation, respectively.

Apart from a similarity regarding the scaling on the injected power P_{in} , the agreement between the two expressions is poor, a major difference being the difference in scaling with the plasma current governed by the strong dependence of the λ_q -scaling on the safety factor. Using the scaling of τ_E , one can also determine that of λ_q :

$$\lambda_q^{E,L} = q^{0.99} P^{0.26} n^{-0.13} M^{-0.07} R^{0.07} A^{-0.05} k^{-0.63} \quad (33b)$$

As readily expected, this scaling law exhibits a weaker dependence on the safety factor q , and a similar scaling on the injected power P_{in} . The other engineering scaling factors appear to have a small impact but for the elongation k at given safety factor. Plasma shaping appears to enhance confinement (33a) which consequently drives a reduction of the SOL width (33b).

In the framework of the κ - ε approach, one can also estimate the energy confinement time given the diffusion coefficient D (25) stemming from κ and ε . The latter are governed by the the closure constraint on the SOL width that is used to determine V .

For the chosen scaling relationship of D_V (26), one can estimate the confinement time $\tau_E^{(\kappa-\epsilon)}$ considering that $\tau_E^{(\kappa-\epsilon)} \propto a^2/D_V$:

$$\Omega_0 \tau_E^{(\kappa-\epsilon)} \propto \rho_*^{-1-2\epsilon_\rho} q^{-\epsilon_q} A^{-\epsilon_A} \quad (34a)$$

The Larmor pulsation $\Omega_0 = eB/m$ is used to enforce the standard time normalization. The values obtained for the different exponents ϵ_ρ , ϵ_q and ϵ_A for the different scaling laws and for the κ - ϵ model are summarized in Table 2. As shown in Appendix B, the adimensional parameter form of the $\tau_{E,th}^L$ scaling is:

$$\Omega_0 \tau_{E,th}^L \propto \rho_*^{-1.85} q^{-3.74} A^{0.37} \beta^{-1.41} \nu_*^{0.19} k^{3.22} \quad (34b)$$

The expression of ν_* used here is the collision frequency normalized by the passing particle transit frequency $1/\tau_{\parallel}$. Therefore, it does not contain the aspect ratio dependence governed by the trapped particles physics so that ν_* is proportional to A . Because of this difference in definition, the scaling exponent on the aspect ratio is different from that of the ITER physics basis [91]. Comparing (34a) and (34b) then yields the condition $\epsilon_\rho = 0.425$, $\epsilon_q = 3.74$ and $\epsilon_A = -0.37$ to match the power law dependencies. For the two characteristic times addressed above, one finds that the choice $\tau \propto 1/\gamma_I$ discussed at the beginning of this Section departs less from the present results than $\tau \propto \tau_{\parallel}$ used in Section 2.2. While the transport governed by the local diffusion coefficient derived in the κ - ϵ framework is typically GyroBohm $\epsilon_\rho = 1$, the value obtained to match the L-mode scaling law is not even Bohm-like $\epsilon_\rho < 1/2$. The β dependence also leads to a significant difference. However, it is to be mentioned that the dependence on β is presently being questioned [102]. Finally, the ν_* dependence is relatively small and has therefore a rather weak effect. Given the differences, in particular regarding the β and ρ_* scaling, it is interesting to compare the κ - ϵ expression of τ_E (34a) to the latest analysis of the empirical data base of the energy confinement time [102].

$$\Omega_0 \tau_{E,th}^H \propto \rho_*^{-3.0} \beta^{0.0} \nu_*^{-0.14} q^{-1.7} k^{3.22} A^{0.9} \quad (34c)$$

This expression is slightly different from that in Ref.[102] because in this reference ν_* includes the aspect ratio dependence of the trapped particle physics but does not appear to include the dependence on the safety factor. We have thus modified the scaling law to take into account the differences in the definitions of ν_* . This scaling law being GyroBohm, one readily finds $\epsilon_\rho = 1$. One can also identify $\epsilon_q = 1.7$ and $\epsilon_A = -0.9$ that are somewhat closer to the values obtained with $\tau \propto 1/\gamma_I$, $\epsilon_q = 2$, $\epsilon_A = -0.5$, than with $\tau \propto \tau_{\parallel}$, $\epsilon_q = 1$, $\epsilon_A = -1$, see Table 2. This is when having in mind that the dependence on the safety factor is based on a more complete experimental evidence than that on the aspect ratio. The fact that the new scaling law of the energy confinement time does not depend on β , makes the connection between energy confinement time and SOL width more relevant since the SOL width does not appear to depend on β .

The 1D model allows one comparing the predicted energy confinement time to that

<i>scaling law</i>	ϵ_ρ	ϵ_q	ϵ_A
<i>ITER – L</i> [91]	0.425	3.74	0.37
<i>ITER – H</i> [102]	1	1.7	–0.9
$\kappa - \varepsilon, \tau \propto 1/\gamma_I$	1	2	–0.5
$\kappa - \varepsilon, \tau \propto \tau_{\parallel}$	1	1	–1

Table 2. Given the definition $D = ac_s \rho_*^{2\epsilon_\rho} q^{-\epsilon_q} A^{-\epsilon_A}$, see (26), leading to (34a), values of the exponents $\epsilon_\rho, \epsilon_q, \epsilon_A$ for the ITER confinement time scaling laws compared to values obtained with the κ - ε model assuming $\lambda/a \propto q\rho_*$ (22b).

achieved in the steady state 1D κ - ε simulations. The appropriate trend of the dependence on the input power P_{in} is recovered, Figure 10 left hand side. The power law fitting the numerical results, black open squares, is $P_{in}^{-0.64}$ blue short-dash line, comparable to $P_{in}^{-0.73}$ given by the ITER L-mode scaling (33a), dashed black line. One finds that the simulation data also compares well with the power law (32b) stemming from the empirical SOL width scaling $P_{in}^{-0.66}$, dash-dot black line. The gyroBohm scaling of transport used for the κ - ε model, hence for $\epsilon_\rho = 1$ in (34a), yields $P_{in}^{-0.6}$ blue long-dash line. The scan in B_{pol} , hence in I_p appears to qualitatively right, Figure 10 right hand side but the quantitative power law fitting leads to more discrepancy. The model data, open black squares, is fitted by the power law $B_{pol}^{0.98}$ short dash blue line, which is quite close to $B_{pol}^{0.96}$ given by the ITER L-mode scaling (33a). However, the power law computed using various closures do not give such a good match. Using the SOL width scaling law one obtains $B_{pol}^{2.2}$ (32b) black dash-dot line. The theory based κ - ε models (34a) yield $B_{pol}^{0.8}$ for $\epsilon_q = 2$ long-dash blue line $B_{pol}^{0.4}$ for $\epsilon_q = 1$ dotted blue line. For such models the perfect fit would be obtained with $\epsilon_q = 2.4$. One finds therefore that the characteristic time that must be considered to recover the turbulent transport properties appears to be more difficult to define and understand than the properties governed by the characteristic amplitude of the fluctuating velocity.

3.4. Including turbulence shearing by large scale flows

The proposed κ - ε model for plasma turbulent transport is closely linked to that used in neutral fluids. In particular, the field ε governs the energy cascade providing conditions for steady state turbulent spectra. However, a key physics mechanism that controls plasma turbulent transport is eddy shearing by large scale flows, typically the zonal flows [103]. In particular, the regimes of improved confinement reported in magnetically confined plasmas are observed to be governed by turbulence shearing by large scale flows, see Ref.[104] and references therein. A means to include such a mechanism is to introduce a second predator of the energy of the turbulence, typically the zonal flows [51, 103, 58, 52]. Alternatively, we propose to take into account the shear stabilization in a simplified way.

The previous Sections indicate that the physics enforced by the κ - ε model of this paper can be addressed in terms of the effective microscopic velocity V that governs

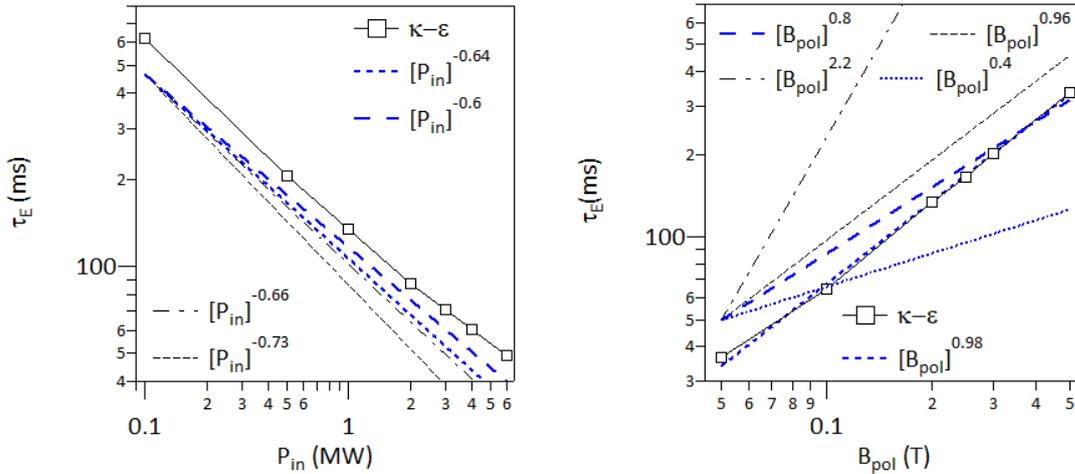


Figure 10. Comparison of the power law dependence of the thermal energy confinement time τ_E on the injected power P_{in} , Left hand side, and on the plasma current I_p given $B_{pol} \propto I_p$, Right hand side. The 1D simulation data, open black squares is fitted with the power law plotted with blue short-dash lines. The ITER L-mode scaling law (33a) yields the black dashed line. The κ - ε scaling stemming from the empirical scaling law of the SOL width (32b) is plotted with dash-dot black lines while the theory based κ - ε models (34a) are plotted with blue long-dash lines for $\epsilon_q = 2$ and blue dotted line for $\epsilon_q = 1$. Note that for the injected power scaling law, these two cases yield the same behavior.

the convective transport at the smallest scales. As discussed, this velocity is found to be of the order of magnitude as the $E \times B$ drift velocity, thus proportional to $c_s \rho_*$. We now change the definition of V to take into account the large scale velocity shear effect. Let ω_z be the radial shear of the zonal flows and let τ_z be the characteristic time to enforce the shearing effect, the new dimensionless control parameter is then $(\omega_z \tau_z)^2$ since the shearing stabilization does not depend on the direction of the large scale flow, and therefore does not depend on the sign of ω_z . We then write the modified expression of V as:

$$V = c_s \rho_* V_0 \frac{1 + (\omega_z \tau_z)^2}{1 + r(\omega_z \tau_z)^2} \frac{q^{\eta_q}}{A^{\eta_A}} G(\Theta)^{1/2} \quad (35a)$$

The parameter r governs the ratio between the two asymptotic limits, the small shear limit $(\omega_z \tau_z)^2 \rightarrow 0$, such that the term standing for the shearing effect is equal to 1, and the large shear limit, $(\omega_z \tau_z)^2 \rightarrow +\infty$, such that this factor is equal to $1/r$. The values $r > 1$ govern a reduction of the turbulent energy amplitude. The proposed dependence of V on the shearing efficiency $(\omega_z \tau_z)^2$ is devised as a simple expression that governs a reduction of V by the factor r between the no-shear asymptotic limit and the strong shear asymptotic limit. The fixed point for κ - ε is such that $\kappa = V^2$ and $\varepsilon = \gamma_I \kappa$ so that

$D = \kappa/\gamma_I$. Given $1/\gamma_I = a\sqrt{A}/c_s$, the modified diffusion coefficient is then:

$$\begin{aligned} D &= \frac{V^2 a \sqrt{A}}{c_s} = \frac{c_s^2 \rho_*^2 a \sqrt{A}}{c_s G(\Theta)} \left[\frac{1 + (\omega_z \tau_z)^2}{1 + r(\omega_z \tau_z)^2} \right]^2 \frac{q^{2\eta_q}}{A^{2\eta_A}} \\ &= a c_s \rho_*^2 V_0^2 \left[\frac{1 + (\omega_z \tau_z)^2}{1 + r(\omega_z \tau_z)^2} \right]^2 \frac{q^{2\eta_q}}{A^{2\eta_A - 0.5}} \end{aligned} \quad (35b)$$

$$\lambda = a q \rho_* \left[\frac{1 + (\omega_z \tau_z)^2}{1 + r(\omega_z \tau_z)^2} \right] \frac{q^{\eta_q - 1/2}}{A^{\eta_A - 3/4}} \quad (35c)$$

In the 1D simulation case the parameter ω_z is defined given the electric potential profile solution of the vorticity equation (A.1b). Two free parameters then remain to be fixed, τ_z that governs the sensitivity to the shearing mechanism and r that governs the amplitude of the transition from the low to the high shear limits. For a factor two change in the diffusion coefficient, one must choose $r \approx \sqrt{2}$. For the more important variation between the L and H mode characterized by a decrease of the SOL width by a factor 2 to 3, as discussed previously for the parameter α_s , one must consider $r \geq 2$. However, the modification of the ionization source in the core plasma, and consequently modified density gradients in the edge and SOL region also contribute to the changes of κ and ε making the overall effect more complicated than the argument solely based on the value of the correction of V due to the flow shear. The modified model is found to undergo

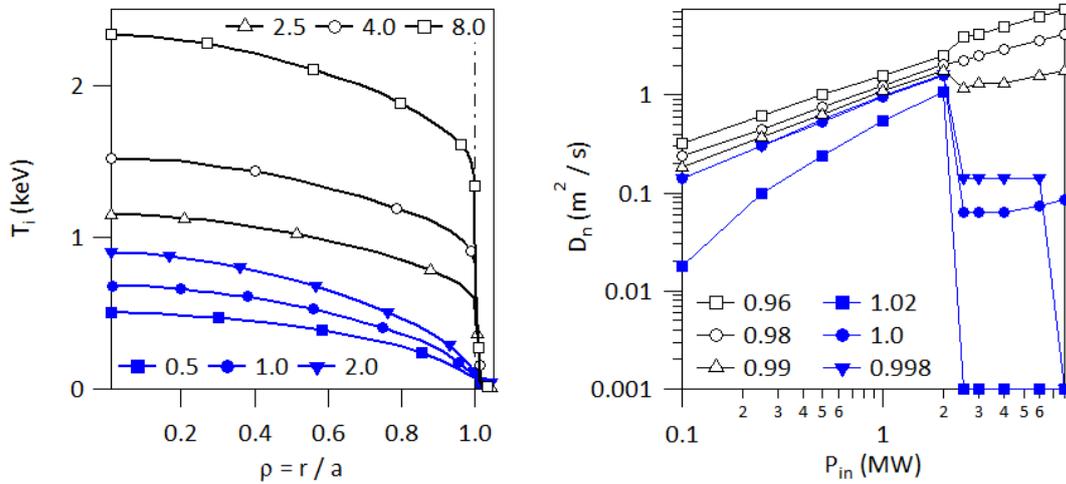


Figure 11. Stabilization by $E \times B$ shear. Left hand side: Radial profiles of the ion thermal energy T_i when ranging the input power P_{in} from 0.5 MW to 8 MW, closed square $P_{in} = 0.5$ MW, close circle $P_{in} = 1$ MW, closed head down triangle $P_{in} = 2$ MW, open head up triangle $P_{in} = 2.5$ MW, open circle $P_{in} = 4$ MW and open square $P_{in} = 8$ MW. Right hand side: variation of plasma diffusivity D_n with input power P_{in} at different radial positions. Within the separatrix: open square $\rho = 0.96$; open circle $\rho = 0.98$, head up open triangle $\rho = 0.99$, closed head down triangle open square $\rho = 0.998$, and outside the separatrix: closed circle open square $\rho = 1.0$, open square $\rho = 1.2$.

a transition from the no-shear regime to the strong shear regime when the injected

power is increased Figure 11. The effect of shear appears to be localized just inside the separatrix, close to the inversion point of the electric field Figure 11 left hand side. Indeed, the profiles appear to undergo a transition between the low power regime, closed symbols, and the high power regime, open symbols. Steepening of the ion thermal energy gradient is observed in the vicinity of the separatrix, a behavior that is reminiscent of an interface barrier [59]. The mechanism driving the shear layer is the transition from the core where the electric potential is typically proportional to $-T_i/e$ to the SOL region where the sheath boundary condition lead to an electric potential proportional to T_e/e . The increase of the electric field in the vicinity of the separatrix is then governed by enhanced temperature gradients in the edge when increasing P_{in} . When plotting the variation of the plasma diffusion coefficient D_n with injected power P_{in} , Figure 11 right hand side, One can observe that for the inner most radial position $\rho = 0.96$, D_n increases monotonically with P_{in} and as the position gets closer to the separatrix, a decrease of D_n sets in for $2 MW \leq P_{in} \leq 2.5 MW$. The enhanced gradient is driven both by the heating source and by the decrease of the turbulent transport D_n driven by the velocity shear. The latter depending on the gradients as well as on turbulent transport provides a mechanism for a nonlinear feedback process, and therefore a transition to improved confinement driven by the heating power.

The modification of turbulent transport governed by the flow shear we have presented here is an ad hoc way of generating a feedback on turbulence and governing a further process of self-organization of plasma transport. The transition to barrier formation could then be modeled self-consistently provided the free parameters can be determined by matching the simulations to experimental evidence. It is to be underlined that the chosen expression of V determines the impact of the flow shear on transport. It is devised to govern a reduction of the cross-field diffusive transport. The observed transition follows the standard feedback loop where the reduction of transport drives steeper gradients that in turn enhance the shear stabilization [104]. However, the reduced model used here does not account for an initial step enforced by Reynolds's stress generation of zonal flows [105, 106]. Alternative reduced models governing the local behavior of κ and ε driven by this physics have been proposed to investigate the transition to transport barriers [51, 56, 58]. It is to be noted that in the ad hoc model proposed here, a modest change in the magnitude of V with a monotonic dependence on the shear, is enough to drive a bifurcation behavior. This result must be confronted to experimental evidence and matching parameters tuned to obtain a universal behavior. If this can be achieved, this minimal model could be used to investigate the compatibility of high performance divertor regimes preserving the H-mode confinement in the core.

4. SolEdge2D-Eirene modeling of a WEST discharge

This Section introduces the first results provided by the 2D κ - ε model of a WEST experiment [107, 45] in the lower-single-null configuration, discharge #55049. The transport code SolEdge2D-Eirene [11] is used for pure deuterium plasma (no impurity

physics), and without the mean field velocity drifts. The equations are recalled in Appendix C. The coefficients of the diffusive and conductive perpendicular fluxes (D_n , χ_e , χ_i and ν) are self-consistently estimated using the κ - ε model, Section 2. The equations are discretized using a second-order finite volume scheme associated to a volume penalization technique [108, 109] on a structured mesh based on a grid aligned on the magnetic flux surfaces for numerical efficiency, Fig. 12. Typically 10^6 iterations

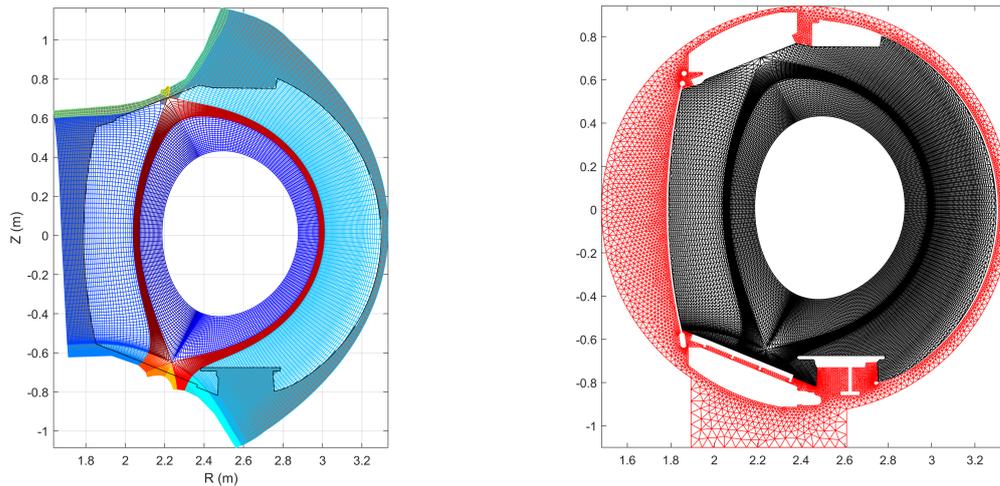


Figure 12. Left hand side: Example of mesh decomposition for a WEST magnetic equilibrium with double X-point [107, 45]. Each sub-domain is characterized by a different color. The penalization technique allows one to add an axisymmetric object, such as a baffle or a toroidal secondary limiter within this computational domain and investigate its impact on plasma-wall interaction. Right-hand side: corresponding mesh for Eirene, overlapping the plasma domain in black, and in regions shielded from the plasma in red within the circular vacuum vessel.

are needed to reach steady state conditions with, as well as without, the κ - ε model. The plasma parameters for this WEST discharge as well as the control parameters of the simulation are presented in Table 3 and Table 4, respectively.

As expected from the chosen dependence of the growth rate (19), one finds that the SolEdge2D-Eirene simulation of the WEST shot 55049 exhibits a clearly ballooned pattern for the field κ localized on the low-field side of the torus, Figure 13 left hand side. Using a logarithmic scale for κ normalized by its maximum value, allows one to highlight the difference of magnitude and to observe spreading governed by parallel transport in the near SOL region of the turbulence energy κ . The far SOL region on the high field side, with near zero plasma pressure, exhibits spurious values of κ , also highlighted by the logarithmic scale. These have very weak effect on the plasma properties. The ballooned pattern is recovered for the diffusion coefficient D , Figure 13 right hand side. In the radial direction, D becomes small in the outer SOL, beyond the secondary separatrix and up to the wall. One can also notice a spreading of D in the

Table 3. Main parameters of the shot *WEST* – 55049. Top line particle and energy control: line averaged density obtained by feedback control \bar{n}_e , additional heating power P_{add} by Lower Hybrid, split onto two antennas, core P_{rad}^{core} and edge P_{rad}^{edge} radiated power. Middle line: magnetic equilibrium parameters, plasma current I_p , toroidal magnetic field B_T , major R and minor a radius. Lower line: dimensionless control parameters, safety factor q_{95} , elongation k , aspect ratio A and X-point height from the divertor floor h_X normalized by the elongation times the minor radius $k a$.

$\bar{n}_e[m^{-2}]$	$P_{add}[MW]$	$P_{rad}^{core}[MW]$	$P_{rad}^{edge}[MW]$
$4.0 \cdot 10^{19}$	4.0	2.0	0.6
$I_p[MA]$	$B_T[T]$	$R[m]$	$a[m]$
0.5	3.6	2.50	0.5
q_{95}	k	A	$h_X/(k a)[m]$
3.4	1.25	5.0	0.132

Table 4. Control parameters for the SolEdge2D-Eirene simulation of the shot *WEST* – 55049, density at the inner boundary n_e^{OMP} , heating power into the simulation domain P_{in} , poloidal field at the outer midplane B_{pol}^{OMP} .

$n_e^{OMP}[m^{-3}]$	$P_{in}[MW]$	$B_{pol}^{OMP}[T]$
$2.5 \cdot 10^{19}$	1.0	0.2

near SOL, governed by parallel advection from the low field towards the high field side. The SolEdge2D-Eirene radial profiles for the density n_e and the electron thermal energy

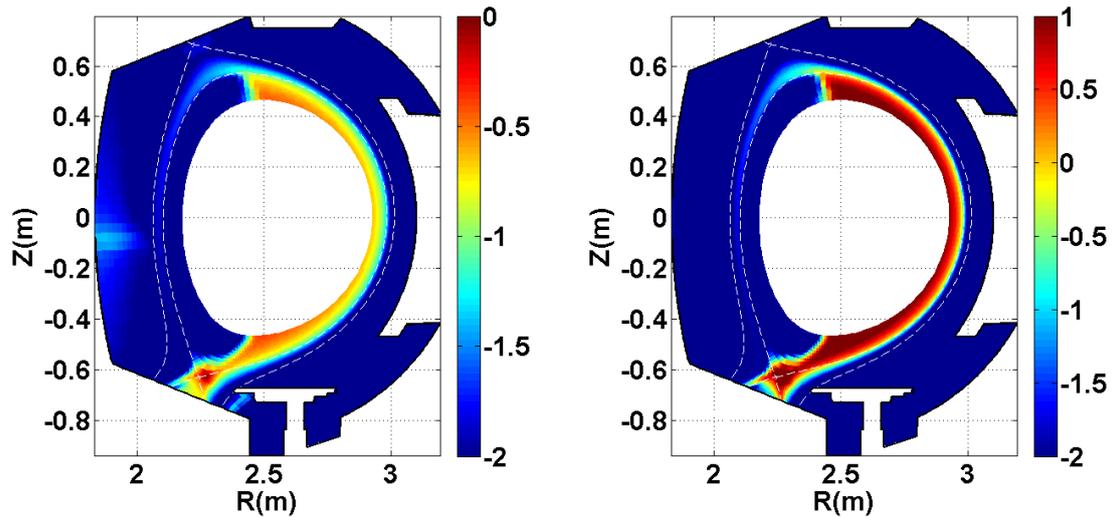


Figure 13. SolEdge2D-Eirene simulation of the *WEST* shot 55049. Left hand side: variation in the poloidal plane of κ normalized by its maximum value with logarithmic scale. Right-hand side: poloidal plane variation of D , units $m^2 s^{-1}$, with logarithmic scale.

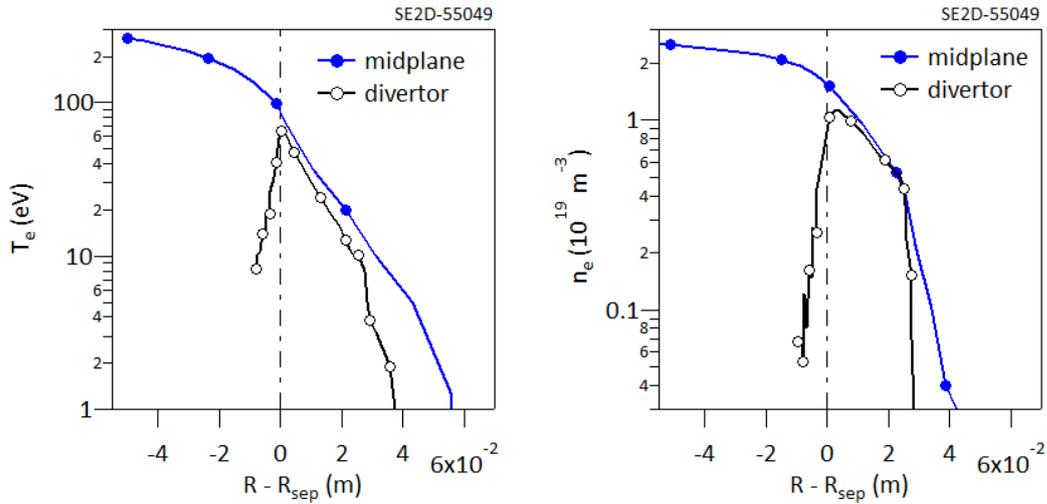


Figure 14. Divertor profiles and radial profiles at the outer midplane of the electron thermal energy, Left hand side, and density, Right hand side, obtained with the SolEdge2D-Eirene simulation of the shot WEST-55049. The divertor profiles are remapped at the midplane. Midplane profile, blue curve closed circles, divertor profile black curve open circles.

T_e are shown on Figure 14 at the outer midplane and at the divertor. A logarithmic scale is used to highlight the SOL part of the profiles. The profiles indicate that the thermal energy and density only weakly vary from the midplane to the divertor that corresponds to the so-called sheath limited or hot divertor regime. Let us notice that in the experiment however one observes a factor two drop in the thermal energy from midplane to the divertor, this is closer to the so-called high recycling regime. The difference appears to be related to the particle recycling pattern and via the boundary condition to a change in the parallel transport behavior. This issue will be further analyzed elsewhere. For the present comparison, the shape of the divertor profiles, which is governed by the cross-field transport and thus the κ - ε model, is more relevant than the actual values. One can notice the sharp drop of the divertor profiles towards the private flux region and slightly stronger gradients in the outer SOL, $R - R_{sep} \geq 0.03$ m.

4.0.1. Midplane profiles The radial density and electron thermal energy profiles are compared to the experimental profiles at the outer midplane, Figure 15. One can notice the agreement between the measurement points and the simulation output in the near SOL region $0.01 \leq R - R_{sep} \leq 0.03$ m. The SolEdge2D-Eirene profile of the electron thermal energy exhibits a too strong gradient, in particular in the outer SOL $0.03 \leq R - R_{sep} \leq 0.05$ m, driving a departure between the simulation profile and the measurements. For the density, the mismatch is restricted to the outer SOL $0.03 \leq R - R_{sep} \leq 0.05$ m where the simulation profile exhibits a too small decay length. These differences are highlighted by the logarithmic scales, and are hardly noticeable with linear scales. Given the fact that there is a single free parameter to tune all cross-field transport in the whole simulation domain the overall agreement is remarkable.

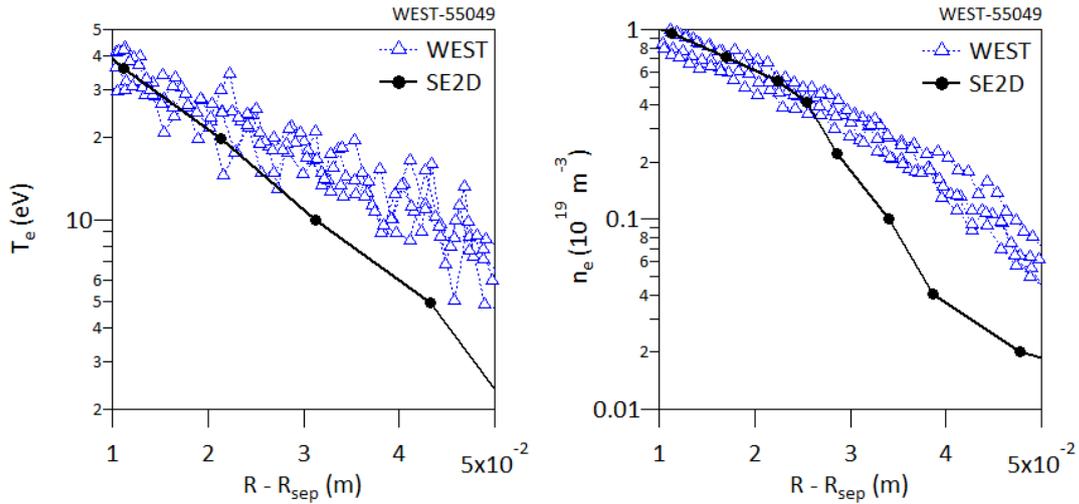


Figure 15. Radial profiles at the outer midplane for the shot WEST-55049. SolEdge2D-Eirene (black curve closed circles), probe data (blue dashed line and open triangles). The experimental data measured with the reciprocating Langmuir probes (RLP) located at the top of the device with vertical plunge are mapped at the midplane. The radial extent is restricted to the region where the experimental data is reliable, in the far SOL $R - R_{sep} \geq 0.05 \text{ m}$, the density is small and the interpretation of the probe characteristics is difficult, when approaching the separatrix $R - R_{sep} \leq 0.01 \text{ m}$ the probe head interacting with a hot plasma starts behaving as a secondary limiter. Left hand side: electron thermal energy. Right hand side: density.

4.0.2. Divertor profiles Figure 16 and Figure 17 show radial divertor profiles, remapped to the midplane in terms of the distance to the separatrix, the SOL profile connected to the midplane for $R - R_{sep} \geq 0$ and for $R - R_{sep} \leq 0$ the profile towards the private flux region. The figures use left and right axis presentation, with on the left hand side axis the scale for the WEST experimental data labeled "WEST", closed blue point dashed line, and on the right hand side axis the scale for the SolEdge2D-Eirene output, labeled "SE2D", black plain line. The difference between these two scales underlines the shortfall regarding the magnitude, while splitting the results between left (experimental) and right (simulation) scales highlights the profile similarities.

On Figure 16, are presented the parallel fluxes to the divertor, the saturation current j_{sat} proportional to the particle flux, left hand side panel, and the parallel heat flux $Q_{||}$ onto the divertor target plate, right hand side panel. The heat flux pattern exhibits comparable magnitude with a peak value for the simulation typically 10 % smaller than the experimental measurement. Agreement regarding the shape is also found with close to exponential fall-off in the SOL region and sharp decay towards the private flux. The e-folding lengths in the SOL are $\lambda_{SOL}^{WEST} = 9.1 \text{ mm}$ and $\lambda_{SOL}^{SE2D} = 7.3 \text{ mm}$, with a difference of order 20 %, and the decay length towards the private flux are $\lambda_{pf}^{WEST} = 1.1 \text{ mm}$, smaller than estimated from the simulation $\lambda_{pf}^{SE2D} = 1.5 \text{ mm}$.

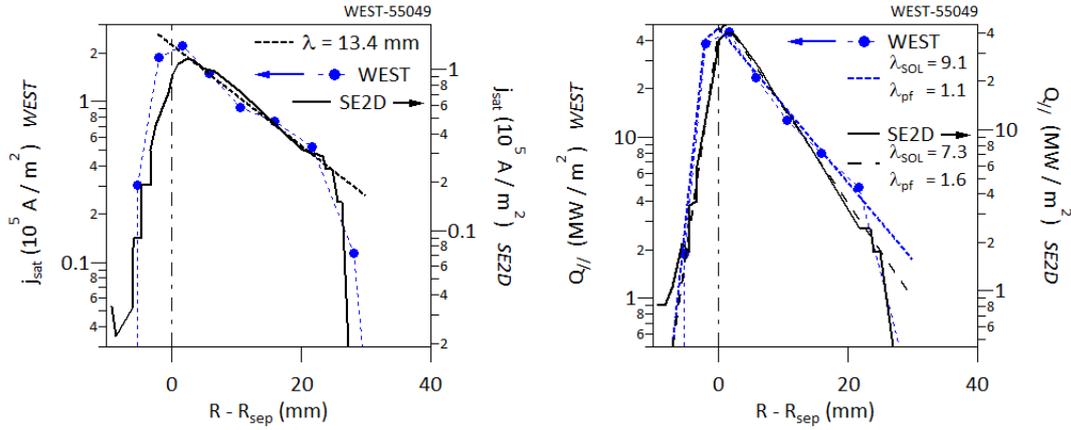


Figure 16. Experimental and simulation divertor profiles remapped at the outer midplane for the shot WEST-55049. Experimental data from Langmuir probes, labeled WEST, embedded in the divertor floor, closed blue circles, SolEdge2D-Eirene plain black line, labeled SE2D. Left and right logarithmic scales have the same ratio between max and min, but are shifted to ensure overlap of the peak values. Left hand side: ion saturation current. Right hand side: parallel heat flux.

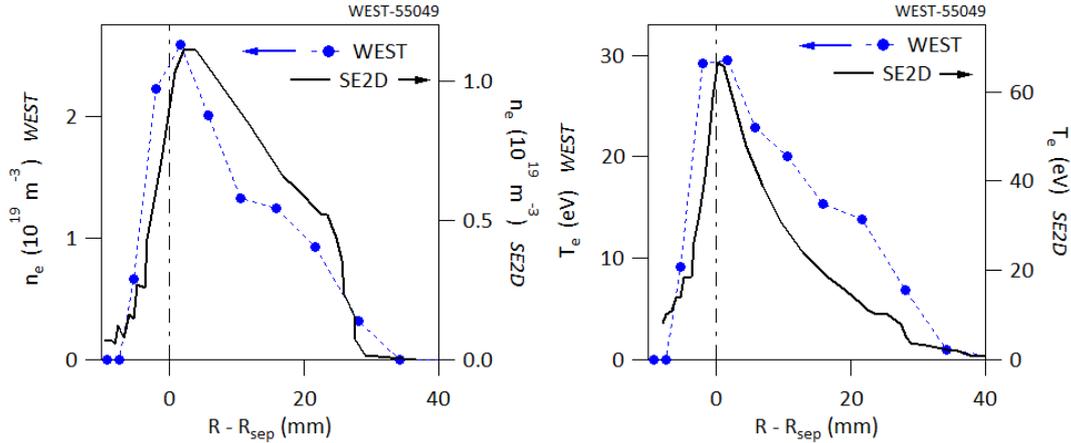


Figure 17. Experimental and simulation divertor profiles remapped at the outer midplane for the shot WEST-55049. Experimental data from Langmuir probes, labeled WEST, embedded in the divertor floor, closed blue circles, SolEdge2D-Eirene plain black line, labeled SE2D. Linear right and left scales are adjusted to ensure the overlap of the maxima. Left hand side: plasma density. Right hand side: electron thermal energy.

Regarding the ion saturation current, the experimental peak value is typically 1.6 times the value achieved in SolEdge2D-Eirene. The simulation is thus characterized by a too large fueling efficiency, and consequently, smaller particle flux to the target plate for the same core density.

Since the electron thermal energy T_e is proportional to the divertor heat flux divided by the particle flux, one expects a factor 1.9 drop of T_e between simulation and experiment, Figure 17 right hand side. In practice one finds a ratio of the order of 2.2, therefore

Table 5. SOL width for WEST: $\lambda_{p_e}^{SE2D}$ from the pressure e-folding length given by SolEdge2D-Eirene in the plasma midplane, $\lambda_{p_e}^{WEST}$ from the pressure e-folding length given by the reciprocating Langmuir probe data in the plasma midplane, from λ_{SOL}^{SE2D} the heat flux channel width in SolEdge2D-Eirene simulation, λ_{SOL}^{WEST} the heat flux channel width from Langmuir probe embedded in the divertor. SOL width prediction for WEST according to available scaling laws, λ_H according to [89], $\lambda_L = 2 \lambda_H$, λ_q according to [87], λ_q^{max} , λ_q^{min} maximum and minimum value according to [87] taking the error bars into account. All values in [mm].

$\lambda_{p_e}^{SE2D}$	$\lambda_{p_e}^{WEST}$	λ_{SOL}^{SE2D}	λ_{SOL}^{WEST}		λ_H	λ_L	λ_q	λ_q^{max}	λ_q^{min}
8.8	7.3	7.3	9.1		4.1	8.2	2.9	18	0.3

reasonably close. One also notices that the profiles shapes do not compare well. Differences in the density n_e profiles, Figure 16 left hand side should behave as the inverse of the thermal energy, and one finds indeed that the density is about 2.3 times larger in the experiment compared to the simulation together with a difference in shape that is the opposite to that seen for the electron thermal energy. Although, there are differences, one finds consistency most likely governed by the conservation of the total plasma pressure along the field lines and by the agreement between simulation and experiment of the plasma energy flux onto the divertor.

Regarding the various ways of determining the SOL width, from e-folding length of the plasma pressure in the midplane, given by SolEdge2D-Eirene λ_{SOL}^{SE2D} , or by the reciprocating Langmuir probe $\lambda_{p_e}^{WEST}$, or from the heat flux deposition pattern, λ_{SOL}^{SE2D} given by SolEdge2D-Eirene, and λ_{SOL}^{WEST} given by the probe data, Table 5 yield quite comparable values 8 ± 0.8 [mm] in the SolEdge2D-Eirene simulations and 8.2 ± 0.9 [mm] for the WEST experiment. This value agrees with the scaling law $\lambda_H = 0.64 B_{pol}^{-1.15}$ [89] for the SOL width in H-mode, with a factor 2 increase from H-mode to L-mode, one obtains $\lambda_L \approx 2\lambda_H = 8.2$ [mm]. The latter value agrees with both simulation and experimental results. Conversely, the scaling law for L-mode [87] leads to λ_q which is about 2.8 times smaller than the observed value. Furthermore, the considerable uncertainty governed the error bars on the exponents leads to a range of values from 0.3 [mm] to 18 [mm]. One finds that the SolEdge2D-Eirene simulation with the κ - ε transport model has better predictive capability than the scaling law built for L-mode data. The latter appears to exhibit a too strong dependence on the safety factor. The uncertainty on the exponents and that for the safety factor is also an issue. Finally, the WEST experiment analyzed here is characterized by a large aspect ratio. The dependence on that parameter is not well known because of the lack of appropriate data in the databases that have been used. As discussed in Section 3.3, a complete picture of cross-field dependence on aspect ratio and safety factor is still missing. This issue is partly made more complicated because of the dependence of the safety factor on the aspect ratio A , $q_{cyl}A = B_T/B_{pol}$, where q_{cyl} is the cylindrical approximation of the safety factor. The open issue appears to be therefore the balance between magnetic field properties B_T/B_{pol} and geometry $A = R/a$.

With the confrontation to the WEST experiment we thus find that the κ - ε cross-field model of transport is robust and compares well with experimental evidence despite the fact that the simulation exhibits differences in the divertor regime, most likely governed by a reduced particle screening capability in the simulation compared to the experiment.

5. Discussion and conclusion

In the present paper we have developed the theoretical background of the κ - ε model, and we have shown its relevance to quite significantly improve the predictive capability of 2D plasma transport models based on averaged fluid equations. Assuming turbulent diffusive transport, the model allows to self-consistently estimate the cross-field fluxes in the edge and SOL regions of diverted plasmas. Instead of tuning by hand the coefficients of perpendicular transport for particles, momentum and heat with some kind of trial process to match as well as possible a given experiment, they are computed at each time and each position from two fields κ and ε governed by two additional transport equations algebraically derived. κ is the turbulence kinetic energy, and ε is interpreted in this paper as the loss via the energy cascade to the energy dissipation scale. One can then define a diffusion coefficient $D = \kappa^2/\varepsilon$ and a characteristic time $\tau = \kappa/\varepsilon$.

While inspired from neutral fluid dynamics, we show that both predator-prey models [50, 51, 52] and quasilinear [60, 61, 62] frameworks used to describe plasma turbulence provide a theoretical background to the κ - ε description of turbulent transport.

The local evolution of each field is governed by a linear term driving exponential transients and a quadratic saturation term. Non vanishing fields are obtained with an interchange like growth rate γ_I . The dissipation contribution in the κ equation is of the form $\kappa^2/D_\omega + \varepsilon$. Consistently with the neutral fluid approach we have used the asymptotic limit $D_\omega \gg 1$ in the simulations so that only ε acts as dissipation of the kinetic energy of the velocity fluctuations. The quadratic dissipation term in the ε equation, of the form $\beta\varepsilon^2$, is defined with $\beta = V/\kappa^{3/2}$, where V is a characteristic velocity. The choice made for the velocity V is governed by the dimensionless scaling of the width of the plasma boundary layer, the SOL width λ_{SOL} that is supposedly the background of well known empirical scaling laws of the SOL width [88, 87]. We have then obtained $V/c_s \propto \rho_* \sqrt{\gamma_I \tau_{\parallel}}$, c_s being the sound velocity, ρ_* the ratio of a characteristic Larmor radius and plasma minor radius a , and $\tau_{\parallel} \propto qAa/c_s$ the parallel connection time, see (22d).

Results of the 1D model are confronted to experimental evidence by analyzing the computed SOL width and comparing the results to the existing scaling law for L-mode plasmas [87]. Since the chosen closure for V accounts for the latter, one could expect close agreement between the empirical scaling law and the 1D model. It is not the case. This has led us to analyze the scaling law and several shortcomings have been identified, the most striking being that its dimensionless form departs significantly from the reference $q\rho_*$ dependence and that in fact it is not even dimensionally correct. One

also finds that the dependence on the safety factor appears to have a too big exponent leading to a too strong dependence on the plasma current. Since the transport model also governs the energy confinement time, we have also compared the $1D$ simulations with scaling law of the energy confinement time, addressing both the dependence in terms of the engineering parameters and the dimensionless aspects. Modification of the scaling exponents of ρ_* , A and q in the expression of V/c_s have been calculated to match the L-mode and H-mode confinement time dependencies. Finally the $1D$ model has been used to illustrate one of the possible problems that can be addressed in the κ - ε framework. Introducing in the definition of the velocity V a dependence on the shear of large scale flows, typically the zonal flows [56], so that V exhibits a monotonic decrease from the low shear limit to the large shear limit, the ratio between these values ranging from $\approx \sqrt{2}$ to ≈ 2 , we have reported in $1D$ simulations an H-mode like transition when increasing the input power, generating an increased stored energy thanks to an interface barrier located at the separatrix [59]. This result is not quite a surprise since the predator-prey models have been introduced in fusion plasmas to investigate such barrier formation [51, 103, 58, 52]. Furthermore, the model has been devised to exhibit the appropriate trend, namely a decrease of V with flow shear. However, the bifurcation aspect is not introduced specifically in the model. The transition thus appears via a monotonic dependence of V on the shear completed by a feedback mechanism which governs a further increase of the flow shear as transport is reduced. This shows that the κ - ε model has the capability to generate changes of the transport regimes in a self-consistent manner.

Stepping to the $2D$ plasma-wall simulations in L-mode, we find that the κ - ε model yields a good match with the experimental profiles, both at the divertor and at the midplane. The transport is shown to be ballooned, as expected for an interchange like instability, driving turbulent transport in the divertor SOL and nearly no transport in the private flux region. The SOL width of WEST is recovered far more precisely than when using the L-mode scaling for the SOL width [87]. It appears in fact that twice the H-mode scaling [89] does a better job in predicting the SOL width. These first simulations with the κ - ε model demonstrate the ability of the model to predict equilibrium profiles at midplane and at the outer divertor target as well as providing a $2D$ distribution of turbulent transport with radial and poloidal dependencies determined self-consistently, in particular the ballooning aspect reported in experiments [29, 30, 31].

We have thus shown the remarkable capability of the κ - ε model to capture key aspects of the physics of turbulent transport throughout the plasma. The present simulations, which are first tests of the model, use a single scalar as tuning parameter to describe the whole $2D$ dependence of the turbulent diffusion coefficient. The confrontation to experimental data is quite convincing, both for midplane and divertor profiles. The width of the energy exhaust channel is also recovered in an L-mode simulation of WEST.

Further adjustment of the model can be achieved using experiments and/or micro-turbulence simulations. The model can be used to model the core plasma as well as self consistent transitions to improved confinement regimes. Indeed, the non-linear dependence of the diffusion coefficients could allow one extending the use of such transport modeling to transients. This modeling effort would no longer be restricted to steady state as the usual diffusive ansatz would imply. Similarly, because the diffusive coefficients governing the transport of the two fields κ and ε are defined to depend non-linearly on their values, typically like κ^2/ε , the transport dynamics can depart significantly from that stemming from fixed diffusion processes. Finally, in the present implementation of the model a single instability drives the turbulent fluctuations on the ion characteristic length scale but the model could be completed using other instability growth rates, and one could also split the model according to fluctuations on either ion or electron scales.

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Appendix A. The 1D transport model

Appendix A.1. The transport equations

The 1D model is obtained by averaging the transport equations Appendix C in the poloidal and toroidal directions. Transport is taken into account by diffusion that replace all convective transport processes at the microscopic level. The diffusion coefficients are chosen as discussed in Section 2.1 equation (18). For the closed magnetic surfaces, this average must be considered as a flux surface average while for the open field lines a standard averaging on the parallel direction is used so that the parallel divergence terms yield contributions that are identified as the outflux onto the wall components. Since the parallel dynamics are ignored, the transport equations are then reduced to the density n and thermal energy evolution $E_i = \frac{3}{2}nT_i$ for the ion and $E_e = \frac{3}{2}nT_e$ for the electron. This simplified model assumes quasineutrality and a low Mach number regime. The Mach number of the ion flow u_i/c_s -where c_s is the sound velocity- is thus assumed to be small on average, hence $\langle E_i + \frac{1}{2}m_i n u_i^2 \rangle \approx \langle E_i \rangle$, and is only taken into account via the boundary condition where it is assumed to be finite and typically of order one. While the Bohm condition enforces $|M| \geq 1$ with respect to the local boundary value of c_s , it is not the case on average $\langle m_i u_i^2 \rangle \ll T_i$. The brackets of the average are dropped in the following to simplify the notations. The plasma transport equations solved in the model are then:

$$\left\{ \begin{array}{l} \partial_t n - \frac{1}{r} \nabla_r (r D_n \nabla_r n) = S_n - H(r-a) \frac{n}{\tau_{\parallel}} \\ \partial_t E_e - \frac{1}{r} \nabla_r (r D_n T_e \nabla_r n + \chi_e n r \nabla_r T_e) = S_E - H(r-a) \frac{n T_e \gamma_e}{\tau_{\parallel}} \\ \partial_t E_i - \frac{1}{r} \nabla_r (r D_n T_i \nabla_r n + \chi_i n r \nabla_r T_i) = S_E - H(r-a) \frac{n T_i \gamma_i}{\tau_{\parallel}} \end{array} \right. \quad (\text{A.1a})$$

In these equations, the function $H(r-a)$ is the Heaviside step function used as a mask that defines the SOL region, $H(r-a) = 0$ for $r < a$ and $H(r-a) = 1$ for $r \geq a$ so that the parallel loss terms apply. For these averaged equations, the convective loss term appear to be governed by the effective SOL confinement time for the particles, τ_{\parallel} , typically of order L_{\parallel}/c_s . The coefficients γ_e and γ_i are reminiscent of that computed for the kinetic sheath transmission, but also take into account effects governed by the relationship between the sheath values of the density and thermal energy and their parallel average. The same remark holds for the transport terms since the average of the product of the local values of χ , n T and ∇T is not equal to the product of the average of these fields. The transport coefficients must therefore be considered as effective. For the heat conductivity, the transport coefficient χ_{eff} to be used is such that $\chi_{\text{eff}} < n \rangle \langle \nabla T \rangle = \langle \chi n \nabla T \rangle$. This issue is not specific of the present model.

The plasma equations are completed by a vorticity equation for $\Omega \equiv m_i \nabla_{\perp} \cdot (e n \nabla_{\perp} \phi / B^2 + \nabla_{\perp} p_i / B^2)$, where the electrostatic potential is ϕ , e the electron charge, and p_i the ion pressure. This equation is derived from the charge balance equation

including the polarization drift current for the ions.

$$\left\{ \partial_t \Omega - \frac{1}{r} \nabla_r (r \nu \nabla_r \Omega) = H(r - a) \frac{1}{\tau_{\parallel}} \left(\frac{e\phi}{T_e} - \Lambda \right) \right. \quad (\text{A.1b})$$

The constraint governed by current loss at the sheath is modified to take the linear form of a restoring force of the electric potential ϕ towards $\Lambda T_e/e$. The steady state solution in the core plasma thus governs $\phi \approx -T_i/e$ while in the SOL one has $\phi \propto T_e/e$, provided $|\nabla T_e| \gg |\nabla p_i|/n$. At the separatrix the viscosity ν will then bridge these two asymptotic behaviors. In the standard use of this 1D model, as well as for the SolEdge2D simulation addressed in the paper, large scale flows are not taken into account in the transport. On the same footing the non-linear drive of the vorticity via the Reynolds's stress, mandatory for micro-turbulence simulations is not included in (A.1b). Solving the vorticity equation (A.1b) is only useful when considering an *ad hoc* modification of the control parameter V governed by the $E \times B$ shear.

A simplified transport model for neutrals is also introduced so that the particle source by ionization S_n , and in particular its location, changes consistently with the plasma parameters. Only the particle balance equation is used for the neutrals with density N_0 , diffusive transport with constant diffusion coefficient D_{N_0} and a feed-back controlled gas injection Φ_{N_0} . In this 1D model, we address an ionizing plasma regime such that the ionization and charge exchange cross sections are comparable. The diffusive nature of neutral transport is not particularly well suited since ionization occurs in a couple of steps, charge exchange being subdominant.

$$\left\{ \partial_t N_0 - \frac{1}{r} \nabla_r (r D_{N_0} \nabla_r N_0) = \Phi_{N_0} H(r - a) - S_n \right. \quad (\text{A.1c})$$

The source term Φ_{N_0} should be localized at the outer wall as well as peaked towards $r = a$ to describe recycling via the private flux region of a divertor or in the vicinity of a limiter tip. We simplify these aspects by taking Φ_{N_0} constant throughout the SOL region, hence of the form $\Phi_{N_0} H(r - a)$.

Finally, the cross-field diffusion terms of the plasma transport equations are determined by the κ - ε coupled equations.

$$\left\{ \begin{aligned} \partial_t \kappa - \frac{1}{r} \nabla_r (r D \nabla_r \kappa) &= \gamma_{\kappa} \kappa - \zeta \kappa^2 - \varepsilon \\ \partial_t \varepsilon - \frac{1}{r} \nabla_r (r D \nabla_r \varepsilon) &= \gamma_{\varepsilon} \varepsilon - \frac{V}{\kappa^{3/2}} \varepsilon^2 \end{aligned} \right. \quad (\text{A.1d})$$

It is to be noted that no parallel convective loss term is retained here to account for the SOL transport to the wall. These would reduce the drive governed by the local dynamics, for instance substituting in the κ equation γ_{κ} by $\gamma_{\kappa} - \sigma_{\kappa}/\tau_{\parallel}$ where σ_{κ} is a constant of order unity. However, a similar parallel loss term occurs on the closed magnetic surfaces for parallel currents, which have a stabilizing effect on the interchange drive. Consequently, a similar correction should be applied to γ_{κ} in the region $r < a$. The control parameters that appear in the 1D model (A.1d) should therefore be understood as effective parameters, as discussed for plasma transport (A.1a) and (A.1b). Although,

the control parameters in (A.1d) should be modified to account for the parallel transport, in practice we have assumed that this effect is small for both fields κ and ε .

Appendix A.2. The sources

The particle source distribution in space depends mostly on the neutral penetration into the plasma. The crude model for neutral transport is tuned to obtain realistic profiles of $S_n = nN_0\langle\sigma v\rangle_i$, where $\langle\sigma v\rangle_i$ is the ionization rate. The latter is computed using an expression of the form $\langle\sigma v\rangle_i \propto x^{1/2}/(x_i + x) \exp(-1/x)$ [110], where $x = T_e/E_i$ and $E_i = 13.6 \text{ eV}$ is the ionization energy. With $x_i = 6$ the maximum of the ionization rate occurs at $x = 10$. Overall, this analytical expression yields a dependence of the ionization rate with the electron temperature that is roughly comparable to published data [111, 112]. The neutral source Φ_{N_0} is set to ensure that the density at the separatrix is maintained equal to 1.10^{19} m^{-3} . This can be understood as a particle injection rate with feedback on the plasma separatrix density. A typical radial profile for the neutral density in this model is displayed on Figure A1 left hand side for the following reference plasma parameters typical of a WEST experiment: $R = 2.5 \text{ m}$, $a = 0.5 \text{ m}$, $B_T = 3.7 \text{ T}$, $B_P = 0.2 \text{ T}$, $P_{in} = 1 \text{ MW}$. One finds that the neutral density decreases from the wall into the plasma. The typical e-folding length is approximately 0.05 m , hence a tenth of the chosen minor radius a . This decay rate is a combined effect of the diffusive transport and of the ionization sink, the latter prevailing in the confined plasma region. The

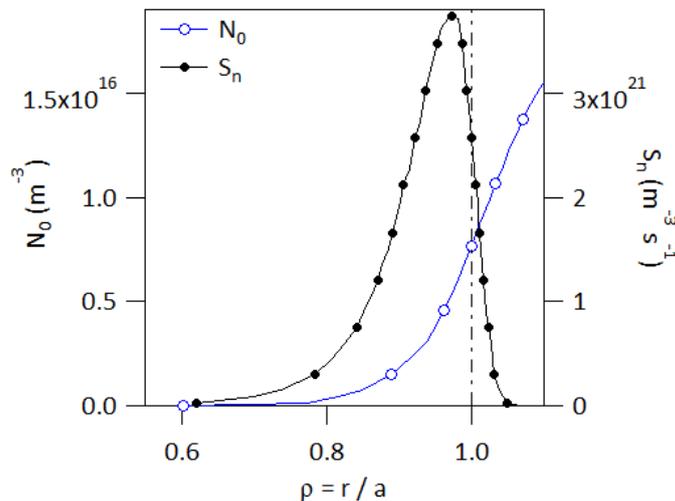


Figure A1. For WEST like parameters $P_{in} = 1 \text{ MW}$, $R = 2.5 \text{ m}$, $a = 0.5 \text{ m}$, $B_T = 3.7 \text{ T}$, Left hand side axis: Radial profile of the neutral density N_0 blue line open circles, Right hand side axis: radial profile of the ionization source term S_n black line closed circles.

source term, Figure A1 right hand side, is peaked close to the separatrix and decays rapidly towards the plasma core. The ionization source in the SOL is localized in the vicinity of the separatrix.

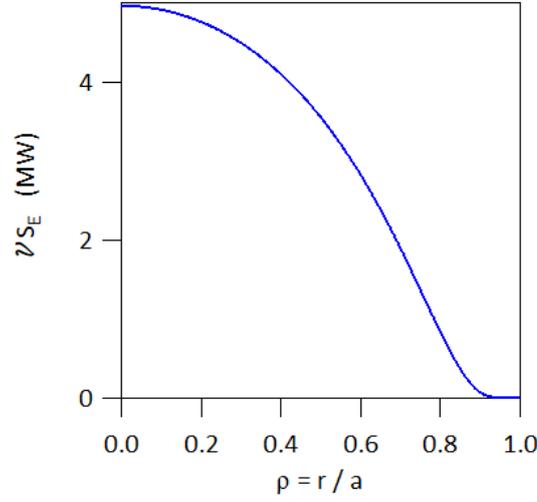


Figure A2. Radial distribution of the energy source $\mathcal{V}S_E$, where \mathcal{V} is the plasma volume and S_E the source of energy density in MW/m^3 .

For a typical plasma volume $\mathcal{V} = 2\pi R\pi k a^2$, where R is the major radius, a the minor radius and k the plasma elongation, the chosen heat source profile S_E , Figure A2, is defined as a function of the normalized radius $\rho = r/a$:

$$\mathcal{V}S_E = \frac{P_{in}}{p_0} \exp\left(-\frac{1}{1-\rho^2}\right) \quad (\text{A.2a})$$

$$p_0 = \int_0^1 \rho d\rho \exp\left(-\frac{1}{1-\rho^2}\right) = 7.425 \cdot 10^{-2} \quad (\text{A.2b})$$

where the constant $p_0 = 7.425 \cdot 10^{-2}$ ensures that the cylindrical integral of $\mathcal{V}S_E$ is P_{in} , the injected power.

Appendix B. Adimensional scaling laws

Appendix B.1. Adimensional scaling of the confinement time

The generic form of the scaling laws that are considered for the energy confinement time is given in terms of the so-called engineering parameters as follows:

$$\tau_E \propto I_p^{\alpha_I} B_T^{\alpha_B} n^{\alpha_n} P^{-\alpha_p} M^{\alpha_M} R^{\alpha_R} A^{\alpha_A} k^{\alpha_K} \quad (\text{B.1})$$

where τ_E is obtained in [s], for I_p , the plasma current in [MA], B_T the toroidal magnetic field in [T], P the loss power in [MW], n the line-averaged density in [$10^{19}m^{-3}$], M the average ion mass in [AMU], R the major radius in [m] and finally A and k the aspect ratio and elongation, respectively. It must be underlined that in fact this

α_I	α_B	α_n	α_p	α_M	α_R	α_A	α_k
0.96	0.03	0.4	0.73	0.2	1.83	0.06	0.64

Table B1. Exponents of the engineering scaling law of the energy confinement time of the *ITER*1996 L-mode scaling.

kind of expression is a mix including dimensional parameters I_p , B_T , n , P and R and dimensionless parameters M , k and A . Some parameters govern the magnetic equilibrium including its geometry, I_p , B_T , R , A , k , while others are less directly controlled such as the density n and the average mass ratio M that are really governed by particle confinement and sources, and the heating power coupled to the plasma P . One can understand such a scaling law either as a fit that holds over a limited range of the parameter values, hence a local tangential fit, or as an effective scaling holding for all values of the parameters. Implicitly, the latter understanding of the underlying similarity appears to be favored since the former tangential form would generically introduce offsets, typically $(1 + G/G_0)^\alpha$ for a given parameter G . For $G \gg G_0$ this does not make any difference but the asymptotic behavior for $G \rightarrow 0$ for $\alpha > 0$, respectively $G \rightarrow \infty$ for $\alpha < 0$ is different, and, more importantly has different implications in terms of the underlying physics. Regarding the latter, a more relevant formulation is that given in terms of dimensionless parameters. For fusion plasmas these are the safety factor q , ν_* the normalized collisionality, β the normalized plasma pressure and ρ_* the normalized ion Larmor radius. If only four dimensionless control parameters exist, then the normalized confinement time $\Omega\tau_E$ must depend on these parameters, hence the 5 dimensional parameters must be expressed as functions of 4 dimensionless parameters.

This generates a constraint, the so-called Kadomtsev similarity constraint [91].

$$\nu_* = \left[\frac{n a}{T^2} \right] [qA] A^{s_\nu} \nu_{*0} \quad ; \quad \nu_{*0} = \frac{(e^2/(4\pi\epsilon_0))^2 n_0 a_0}{T_0^2} \quad (\text{B.2a})$$

$$\beta = \left[\frac{nT}{B^2} \right] \beta_0 \quad ; \quad \beta_0 = \frac{n_0 T_0}{B_0^2/(2\mu_0)} \quad (\text{B.2b})$$

$$\rho_* = \frac{T^{1/2}}{aB} \rho_{*0} \quad ; \quad \rho_{*0} = \frac{\sqrt{m_p T_0}}{e B_0 a_0} \quad (\text{B.2c})$$

$$q = F_q(k, \delta) q_{cyl} \quad ; \quad q_{cyl} = \frac{a B_T}{R B_{pol}} \quad ; \quad B_{pol} = \frac{\mu_0}{2\pi} \frac{I_p}{a k F_I(k, \delta)} \quad (\text{B.2d})$$

In these expressions the quantities with 0 subscript stand for the parameter values used for normalization, most often done implicitly by specifying the units. When ν_* is governed by the physics of the trapped particles $s_\nu = 3/2$, otherwise, as more readily done for the edge region $s_\nu = 0$. The parameter ρ_* is defined with the proton mass m_p , however, regarding the physics of cross-field transport, the effective ion Larmor radius is of interest. One must therefore take this effect into account when addressing the dependence on the effective ion mass M . The safety factor is split into the cylindrical contribution q_{cyl} and two dimensionless functions of the magnetic equilibrium F_q and F_I , that depend typically on dimensionless parameters such as the ellipticity k , the triangularity δ , etc. Given these 3 dimensionless parameters one can express the density n , the product $aB = RB_T/A$, and the square root of the thermal energy $T^{1/2}$.

$$n = \left[\frac{T^2}{a B} \right] \left[\frac{\nu_*/A^{s_\nu}}{qA} \right] [B] \frac{1}{\nu_{*0}} \quad (\text{B.3a})$$

$$a B = T^{1/2} \frac{1}{\rho_*} \rho_{*0} \quad (\text{B.3b})$$

$$T^{1/2} = [B]^{1/5} \left[\frac{qA\beta}{\rho_* \nu_*/A^{s_\nu}} \right]^{1/5} \left[\frac{\rho_{*0} \nu_{*0}}{\beta_0} \right]^{1/5} \quad (\text{B.3c})$$

Given (B.3c), one thus obtains:

$$n = n_0 [B]^{8/5} \left[\frac{\rho_* \nu_*/A^{s_\nu}}{qA} \right]^{2/5} [\beta]^{3/5} \quad (\text{B.4a})$$

$$a B = (aB)_0 [B]^{1/5} \left[\frac{qA\beta}{\rho_*^6 \nu_*/A^{s_\nu}} \right]^{1/5} \quad (\text{B.4b})$$

$$T^{1/2} = T_0^{1/2} [B]^{1/5} \left[\frac{qA\beta}{\rho_* \nu_*/A^{s_\nu}} \right]^{1/5} \quad (\text{B.4c})$$

The various constants that appear in (B.4) lead to the definitions of n_0 , $(aB)_0$ and T_0 . In a similar way one can express the plasma current from the definition (B.2d) of the safety factor:

$$I_p = \frac{2\pi}{\mu_0} \left[\frac{k}{A q_{cyl}} \right] [aB] = I_0 \left[\frac{k}{A q_{cyl}} \right] \left[\frac{qA\beta}{\rho_*^6 \nu_*/A^{s_\nu}} \right]^{1/5} [B]^{1/5} \quad (\text{B.5a})$$

The four dimensional parameters n , (aB) , T , I_p are found to depend on the dimensionless parameters q , ν_* , β and ρ_* as well as on the dimensional parameter B . Finally, one must express the heating power. This engineering parameter is specific insofar that it will depend on the internal energy and on the confinement time for steady state conditions as follows:

$$\begin{aligned} P &= \frac{n T \mathcal{V}}{\tau_E} = \frac{n T}{B^2/(2\mu_0)} \frac{\mathcal{V}}{\Omega \tau_E} B^2/(2\mu_0)\Omega_0 \\ &= \frac{Ak\beta}{\Omega \tau_E} a^3 B^3 F_V \frac{2\pi^2 B_0^2 e B_0}{2\mu_0 m} = P_0 \left[\frac{Ak\beta}{\Omega \tau_E} \right] [a B]^3 \\ &= P_0 \left[\frac{Ak\beta}{\Omega \tau_E} \right] \left[\frac{qA\beta}{\rho_*^6 \nu_* / A^{s_\nu}} \right]^{3/5} [B]^{3/5} \end{aligned} \quad (\text{B.5b})$$

The dimensionless function $F_V(k, \delta)$ is introduced when writing explicitly the plasma volume $\mathcal{V} = \pi a(ka)2\pi R F_V$. The confinement time is normalized here by Ω , where Ω is a reference Larmor pulsation depending on B_T .

$$\Omega \tau_E \propto I_p^{\alpha_I} B^{\alpha_B+1} n^{\alpha_n} P^{-\alpha_p} M^{\alpha_M} a^{\alpha_R} A^{\alpha_A+\alpha_R} k^{\alpha_k} \quad (\text{B.6a})$$

$$\Omega \tau_E \propto k^{\alpha_k} q^{\alpha_q} M^{\alpha_M} A^{\alpha_A} \beta^{\alpha_\beta} \rho_*^{\alpha_\rho} \nu_*^{\alpha_\nu} \quad (\text{B.6b})$$

One can then identify the exponents of the power law dependence of the confinement time on the dimensionless parameters. In particular one obtains a_B :

$$a_B = \frac{(\alpha_I + 8\alpha_n - 3\alpha_p + 5\alpha_B - 4\alpha_R + 5)}{5(1 - \alpha_p)} \quad (\text{B.7a})$$

With the coefficients of the ITER1996th L-mode scaling, Table B1 one obtains $a_B \approx -0.15$ instead of zero. The scaling law is not homogeneous. One also recovers the expressions for the other exponents published in Ref.[113]. The three exponents a_β , a_ν and a_ρ characterize the plasma:

$$a_\beta = \frac{(\alpha_I + 3\alpha_n - 8\alpha_p + \alpha_R)}{5(1 - \alpha_p)} \quad (\text{B.7b})$$

$$a_\nu = \frac{(-\alpha_I + 2\alpha_n + 3\alpha_p - \alpha_R)}{5(1 - \alpha_p)} \quad (\text{B.7c})$$

$$a_\rho = \frac{(-6\alpha_I + 2\alpha_n + 18\alpha_p - 6\alpha_R)}{5(1 - \alpha_p)} \quad (\text{B.7d})$$

while the magnetic equilibrium properties are characterized by:

$$a_k = \frac{(\alpha_I + \alpha_K - \alpha_p)}{1 - \alpha_p} \quad (\text{B.7e})$$

$$a_q = \frac{(\alpha_R - 4\alpha_I - 2\alpha_n - 3\alpha_p)}{5(1 - \alpha_p)} \quad (\text{B.7f})$$

$$a_A = \frac{(5\alpha_A + 6\alpha_R - 4\alpha_I - 2\alpha_n - 8\alpha_p)}{5(1 - \alpha_p)} - s_\nu a_\nu \quad (\text{B.7g})$$

and the main ion species effective mass dependence is:

$$a_M = \frac{\alpha_M}{(1 - \alpha_p)} \quad (\text{B.7h})$$

In the ITER physics basis [91], a correction on the exponent α_R is proposed to recover the constraint $a_B = 0$. In practice any of the coefficients contributing to a_B could be corrected as well as any combination of corrections yielding $a_B = 0$. In fact, the scaling law and the error minimization procedure using the data base should be performed with the constraint $a_B = 0$. In the ITER physics basis this point is not discussed and no information is given on the impact of a change of α_R on the accuracy of the scaling law. Another possibility is to modify directly the exponent α_B that only contributes to a_B . This would imply a change from $\alpha_B = 0.03$ to $\alpha_B = -0.01$. This can appear as a small change, however it might be clearly inconsistent with the data, in particular regarding the change of sign.

Appendix B.2. Adimensional scaling of the empirical SOL width

Let us now consider the scaling law proposed for λ_q in Ref.[87]:

$$\lambda_q = \lambda_{*,0} B_T^{u_B} q^{u_q} P^{u_p} R^{u_R} \quad (\text{B.8a})$$

As for the energy confinement time, this scaling law for λ_q in millimeters, assumes that B_T is given in Tesla, P_{in} in MW, R in meters. In a first step, the expression is rearranged to step towards the dimensionless expression. We then introduce $r_B = u_B + 1 - u_R$, $r_q = u_q$, $r_p = u_p$, $r_a = u_R - 1$, $r_A = u_R$ and therefore:

$$\frac{\lambda_q}{a} = \lambda_{*,0} (B_T)^{r_B} q^{r_q} P^{r_p} (aB)^{r_a} A^{r_A} \quad (\text{B.8b})$$

The coefficients u of the scaling law (B.8a) are given in Table B2.

$\lambda_{*,0}$	u_q	u_B	u_p	u_R
1.44 ± 0.67	-0.8 ± 0.32	1.4 ± 0.67	0.22 ± 0.1	-0.03 ± 0.28

Table B2. Exponents of the engineering scaling law of the SOL width proposed in Ref.[87].

$$\begin{aligned} \frac{\lambda_q}{a} &= (1.44 \pm 0.67) B^{r_B} q^{r_q} A^{r_A} \\ &P_0^{r_p} (\Omega\tau_E)^{-r_p} q^{3r_p/5} A^{8r_p/5} A^{3s_\nu r_p/5} \beta^{8r_p/5} \nu_*^{-3r_p/5} \rho_*^{-18r_p/5} [B]^{3r_p/5} \\ &(a B)_0^{r_a} q^{r_a/5} A^{r_a/5} A^{s_\nu r_a/5} \beta^{r_a/5} \nu_*^{-r_a/5} \rho_*^{-6r_a/5} [B]^{r_a/5} \end{aligned} \quad (\text{B.9a})$$

The dependence on $\Omega\tau_E$ can be expressed directly in dimensionless parameters. It seems more appropriate when doing to use the expression that satisfies the Kadomtsev constraint, hence with $a_B = 0$.

$$\begin{aligned} \frac{\lambda_q}{a} &\propto A^{r_A+8r_p/5+r_a/5} A^{s_\nu(3r_p+r_a)/5} \beta^{8r_p/5+r_a/5} \nu_*^{-3r_p/5-r_a/5} \\ & q^{r_q+3r_p/5+r_a/5} \rho_*^{-18r_p/5-6r_a/5} B^{r_B+3r_p/5+r_a/5} \\ & (k^{a_k} q^{a_q} M^{a_M} A^{a_A} \beta^{a_\beta} \rho_*^{a_\rho} \nu_*^{a_\nu})^{-r_p} \end{aligned} \quad (\text{B.9b})$$

One can then express the dimensionless scaling of the SOL width as:

$$\frac{\lambda_q}{a} \propto k^{\ell_k} q^{\ell_q} M^{\ell_M} A^{\ell_A} \beta^{\ell_\beta} \rho_*^{\ell_\rho} \nu_*^{\ell_\nu} B^{\ell_B} \quad (\text{B.9c})$$

where the exponents are given by:

$$\ell_B = \frac{5r_B + 3r_p + r_a}{5} - r_p a_B \quad (\text{B.10a})$$

$$\ell_\beta = \frac{8r_p + r_a}{5} - r_p a_\beta \quad (\text{B.10b})$$

$$\ell_\nu = \frac{-3r_p - r_a}{5} - r_p a_\nu \quad (\text{B.10c})$$

$$\ell_\rho = \frac{-18r_p - 6r_a}{5} - r_p a_\rho \quad (\text{B.10d})$$

$$\ell_k = -r_p a_k \quad (\text{B.10e})$$

$$\ell_q = \frac{5r_q + 3r_p + r_a}{5} - r_p a_q \quad (\text{B.10f})$$

$$\ell_A = \frac{5r_A + 8r_p + r_a}{5} + s_\nu \frac{3r_p + r_a}{5} - r_p a_A \quad (\text{B.10g})$$

$$\ell_M = -r_p a_M \quad (\text{B.10h})$$

Using the coefficients as given in the scaling one finds again that the scaling law is not homogeneous leading to $\ell_B = 0.16$. The only correction on a single exponent that stands within the error bars of the scaling law is a decrease of the exponent of B from -0.8 to -0.956. In that case one obtains:

$$\frac{\lambda_q}{a} \propto q^{2.15} \rho_*^{0.85} A^{0.03} \beta^{0.46} \nu_*^{-0.17} \quad (\text{B.11})$$

This result departs rather strongly from that assumed to hold $q\rho_*$ and used to determine the parameter V of the κ - ε model (22d). Note that regarding the dependence on the aspect ratio A , we have used the expression of ν_* without the trapped particle effect, hence for $s_\nu = 0$. For $s_\nu = \frac{3}{2}$, the exponent for the aspect ratio scaling is found to be -0.01.

Appendix B.3. Adimensional scaling of the κ - ε SOL width

The parameter scans performed with the 1-D κ - ε model indicate that:

$$\lambda_q \propto B_T^{v_B} B_{pol}^{-v_q} P^{v_p} R^{v_R} \quad (\text{B.12a})$$

v_q	v_B	v_p	v_R
0.844	0.277	0.12	-1.06

Table B3. Exponents of the engineering scaling law obtained with the 1-D κ - ε model.

The coefficients v of the scaling law obtained are given in Table B3. With $B_{pol} = B_T/(Aq)$ one can rewrite this dependence as:

$$\frac{\lambda_q}{a} \propto q^{v_q} B_T^{v_B - v_q + 1 - v_R} A^{v_q + v_R} P^{v_p} (aB)^{v_R - 1} \quad (\text{B.12b})$$

We then introduce $r_B = v_B - v_q + 1 - v_R$, $r_q = v_q$, $r_p = v_p$, $r_a = v_R - 1$, $r_A = v_q + v_R$ and therefore:

$$\frac{\lambda_q}{a} \propto k^{\ell_k} q^{\ell_q} M^{\ell_M} A^{\ell_A} \beta^{\ell_\beta} \rho_*^{\ell_\rho} \nu_*^{\ell_\nu} B^{\ell_B} \quad (\text{B.13})$$

where the exponents are given by (B.10). Using the coefficients as given in Table B3, one finds again that the scaling law is not homogeneous leading to $\ell_B = 1.15$. However, the variation of the density during the various scans, governed by the chosen feedback scheme, and that does not exhibit a power law behavior, can explain this issue.

$$\frac{\lambda_q}{a} \propto q^{0.95} \rho_*^{2.26} A^{-0.48} \beta^{-0.05} \nu_*^{-0.05} B_T^{1.15} \quad (\text{B.14})$$

One finds therefore that the result exhibits a dependence on $B_T \rho_*$, hence typically inverse to the size a , that appears to be spurious. Such a size effect can be related to a relative change of the particle source and can have therefore have an impact on the turbulent transport. One can also observe that the κ - ε 1D model appears to exhibit a too strong dependence on R , see Table B4, which could be the drive for the difference between the $\rho_*^{2.26}$ of the 1D model SOL width scaling and the input $\propto \rho_*$ used to determine V . This difference in scaling law can also be related to the particle source localization and consequently the particular feedback scheme used in the present simulations. For a complete comparison further 1D simulations using a proper database of relevant experiments are required, with the same control scheme of the particle source as in the experiments. Furthermore, uncertainty propagation will be needed to identify the parameters that have a strong impact on the observed difference in the ρ_* dependence, and then address the physics and measurements that are crucial. Finally, this work could also lead us to modify the input assumption used to determine V and its ρ_* dependence to obtain a simulation output matching the experimental database.

Appendix B.4. Dimensional scaling of the SOL width $q\rho_*$

Assuming that the SOL width λ_q/a is of the form:

$$\frac{\lambda_q}{a} \propto q^{\epsilon_q} \rho_*^{\epsilon_\rho} A^{\epsilon_A} \quad (\text{B.15})$$

one can step backwards to determine the expected scaling law in terms of the engineering parameters. Given ρ_*

$$\rho_* \propto \frac{T^{1/2}}{B_T a} \quad (\text{B.16})$$

and:

$$T \propto \frac{P\tau_E}{n\mathcal{V}} \quad (\text{B.17})$$

so that:

$$\rho_* \propto \frac{(P\tau_E)^{1/2}}{(na^3 Ak)^{1/2} B_T a} \quad (\text{B.18})$$

One can also use $q = aB_T/(AI_p)$ to obtain:

$$\frac{\lambda_q}{a} \propto \left(\frac{aB_T}{AI_p}\right)^{\epsilon_q} \left(\frac{(P\tau_E)^{1/2}}{(na^3 Ak)^{1/2} B_T a}\right)^{\epsilon_\rho} A^{\epsilon_A} \quad (\text{B.19})$$

Given the expression of τ_E (B.1), one then has:

$$\frac{\lambda_q}{a} \propto \left(I_p^{\alpha_I} B_T^{(\alpha_B-2)} n^{(\alpha_n-1)} P^{(1-\alpha_p)} M^{\alpha_M} a^{(\alpha_R-5)} A^{(\alpha_A+\alpha_R-1)} k^{(\alpha_K-1)}\right)^{\epsilon_\rho/2} \left(\frac{aB_T}{AI_p}\right)^{\epsilon_q} A^{\epsilon_A} \quad (\text{B.20a})$$

which then yields the scaling law of the SOL width given in terms of engineering parameters:

$$\lambda_q \propto I_p^{\alpha_I \epsilon_\rho/2 - \epsilon_q} B_T^{(\alpha_B-2)\epsilon_\rho/2 + \epsilon_q} P^{(1-\alpha_p)\epsilon_\rho/2} n^{(\alpha_n-1)\epsilon_\rho/2} a^{1+(\alpha_R-5)\epsilon_\rho/2 + \epsilon_q} A^{\epsilon_A - \epsilon_q + (\alpha_A + \alpha_R - 1)\epsilon_\rho/2} M^{\alpha_M \epsilon_\rho/2} k^{(\alpha_K-1)\epsilon_\rho/2} \quad (\text{B.20b})$$

Let us consider the case $\epsilon_\rho = \epsilon_q = 1$, $\epsilon_A = 0$ that corresponds to the $q\rho_*$ SOL width scaling:

$$\lambda_q \propto I_p^{\alpha_I/2-1} B_T^{(\alpha_B-2)/2+1} P^{(1-\alpha_p)/2} n^{(\alpha_n-1)/2} a^{1+(\alpha_R-5)/2+1} A^{-1+(\alpha_A+\alpha_R-1)/2} M^{\alpha_M/2} k^{(\alpha_K-1)/2} \quad (\text{B.21a})$$

For the L-mode confinement scaling, Table B1, this leads to:

$$\lambda_q \propto I_p^{-0.52} B_T^{0.015} P^{0.135} n^{-0.3} a^{0.415} A^{-0.555} M^{0.1} k^{-0.18} \quad (\text{B.21b})$$

Since $B_{pol} \propto I_p/a$ and $A = R/a$ one can rewrite this result to obtain an expression comparable to (B.12a).

$$\lambda_q \propto B_{pol}^{-0.52} B_T^{0.015} P^{0.135} R^{-0.555} n^{-0.3} a^{0.45} M^{0.1} k^{-0.18} \quad (\text{B.21c})$$

These results can be compared to those obtained with the scans performed with the 1-D κ - ε model, see Table B4. The agreement is rather poor, but at least there is no disagreement regarding the sign of the exponents. As discussed previously, the feedback process, leading to a change of the core density at fixed separatrix density, is an issue since the density n , assumed to be an independent engineering parameter changes during each of the scans that have been performed.

exponents	v_q	v_B	v_p	v_R
κ - ϵ model	0.844	0.277	0.12	-1.06
expected scaling	0.52	0.015	0.135	-0.555

Table B4. Exponents of the engineering scaling law obtained with the 1-D κ - ϵ model compared to its expected scaling.

Appendix C. The SolEdge2D model

The model [11] is typically a system of Braginskii drift-reduced fluid equations [114] that govern the evolution of the plasma density n , the parallel momentum and the total energy temperature for both electrons and ions assuming *quasi-neutrality* $n_e = n_i$ and *ambipolarity* $v_{\parallel,e} = v_{\parallel,i}$. The latter constraint can be relaxed but the charge balance equation must then be solved to determine the electric field and consequently take into account the large scale $E \times B$ convective transport. In the simplified case with ambipolarity, the equations write as follows:

$$\frac{\partial n}{\partial t} + \nabla \cdot (nv_{\parallel} \mathbf{b} + n\mathbf{v}_{\perp}) = \nabla \cdot [D_n \nabla_{\perp} n] + S_n \quad (\text{C.1a})$$

$$\begin{aligned} \frac{\partial(nv_{\parallel})}{\partial t} + \nabla \cdot [nv_{\parallel} (v_{\parallel} \mathbf{b} + \mathbf{v}_{\perp})] \\ = -\nabla_{\parallel} \left(\frac{nT_i}{m_i} \right) + \frac{q_i n E_{\parallel}}{m_i} + \frac{\mathcal{R}_{ei}}{m_i} + \nabla \cdot (\nu n \nabla_{\perp} v_{\parallel}) + S_{nv} \end{aligned} \quad (\text{C.1b})$$

$$\begin{aligned} \frac{\partial}{\partial t} \left(\frac{3}{2} n T_i + \frac{1}{2} m_i n v_{\parallel}^2 \right) + \nabla \cdot \left[\left(\frac{5}{2} n T_i + \frac{1}{2} m_i n v_{\parallel}^2 \right) (v_{\parallel} \mathbf{b} + \mathbf{v}_{\perp}) \right] \\ = \nabla \cdot \left[\kappa_i \nabla_{\parallel} T_i \mathbf{b} + \chi_i n \nabla_{\perp} T_i + \nu n \nabla_{\perp} \left(\frac{1}{2} m_i v_{\parallel}^2 \right) \right] \\ + q_i n v_{\parallel} E_{\parallel} + \mathcal{R}_{ei} v_{\parallel} + \mathcal{Q}_{ei} + S_{Ei} \end{aligned} \quad (\text{C.1c})$$

$$\begin{aligned} \frac{\partial}{\partial t} \left(\frac{3}{2} n T_e \right) + \nabla \cdot \left[\left(\frac{5}{2} n T_e \right) (v_{\parallel} \mathbf{b} + \mathbf{v}_{\perp}) \right] \\ = \nabla \cdot \left(\kappa_e \nabla_{\parallel} T_e \mathbf{b} + \chi_e n \nabla_{\perp} T_e \right) \\ - e n v_{\parallel} E_{\parallel} - \mathcal{R}_{ei} v_{\parallel} - \mathcal{Q}_{ei} + S_{Ee} \end{aligned} \quad (\text{C.1d})$$

where $\nabla_{\parallel} = \vec{b} \cdot \vec{\nabla}$ and $\vec{\nabla}_{\perp} = \vec{\nabla} - \vec{b} \cdot \nabla_{\parallel}$ define gradients along the parallel and perpendicular direction, $\vec{b} = \vec{B}/B$ being the unit vector along the magnetic field defining the parallel direction. $\mathbf{v}_{\perp} = \mathbf{v} - v_{\parallel} \mathbf{b}$ is the velocity component perpendicular to the magnetic flux surfaces due to the large scale flows when determined (for the present simulations $\mathbf{v}_{\perp} = 0$), E_{\parallel} is the parallel component of the electric field, \mathcal{R}_{ei} is the parallel electron-ion friction force, S_n , S_{nv} , S_{Ei} and S_{Ee} are the particle, momentum and energy sources respectively.

The boundary conditions are derived from the physics of the sheath: at the sheath

entrance, the absolute value of plasma parallel velocity along the magnetic field lines reaches minimum sound speed. This is the *Bohm condition* for the plasma parallel velocity, expressed by $|v_{\parallel}| \geq c_s = \sqrt{\frac{e(T_e+T_i)}{m_i}}$. The latter is used in SolEdge2D as immersed boundary condition in the penalization technique. The parallel heat flux impinging on the solid target $q_{\parallel,BC}$ will read $q_{\parallel,BC} = (\gamma n T v_{\parallel})_{BC}$, where γ is the total sheath transmission coefficient.

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